



Nuclear Group
P.O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

March 8, 1993
ND3MNO:3424

Beaver Valley Power Station, Unit No. 2
Docket No. 50-412, Licensee No. NPF-73
LER 93-003-00

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 93-003-00, 10 CFR 50.73.a.2.iv, "Steam Generator Blowdown Isolation on High Blowdown Tank Level."

F.D. Schuster
for LRF
L. R. Freeland
General Manager
Nuclear Operations

DAW/sl

Attachment

9303110344 930308
PDR ADOCK 05000412
S PDR

IF 2/2
1/1

March 8, 1993
ND3MNO:3424
Page 2

cc: Mr. T. T. Martin, Regional Administrator
United States Nuclear Regulatory Commission
Region 1
475 Allendale Road
King of Prussia, PA 19406

Mr. G. E. Edison, BVPS Licensing Project Manager
United States nuclear Regulatory Commission
Washington, DC 20555

Larry Rossbach, Nuclear Regulatory Commission,
BVPS Senior Resident Inspector

J. A. Holtz, Ohio Edison
76 S. Main Street
Akron, OH 44308

Larry Beck
Centerior Energy
6200 Oak Tree Blvd.
Independence, OH 44101-4661

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

G. E. Muckle,
Factory Mutual Engineering
680 Anderson Drive #BLD10
Pittsburgh, PA 15220-2773

Mr. Richard Janati
Department of Environmental Resources
P.O. Box 2063
16th Floor, Fulton Building
Harrisburg, PA 17120

Director, Safety Evaluation & Control
Virginia Electric & Power Co.
P.O. Box 26666
One James River Plaza
Richmond, VA 23261

March 8, 1993
ND3MNO:3424
Page 3

W. Hartley
Virginia Power Company
5000 Dominion Blvd.
2SW Glenn Allen, VA 23060

J. M. Riddle
Halliburton NUS
Foster Plaza 7
661 Anderson Drive
Pittsburgh, PA 15220

Bill Wegner, Consultant
23 Woodlawn Terrace
Fredricksburg, VA 22405

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Beaver Valley Power Station Unit 2

DOCKET NUMBER (2)

0 5 0 0 0 4 1 2 1 OF 0 3

PAGE (3)

TITLE (4)

Steam Generator Blowdown Isolation on High Blowdown Tank Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)														
0	2	0	5	9	3	9	3	0	0	3	0	0	0	3	0	8	9	3	N/A	0	5	0	0	0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																					
1			20.402(b)			20.405(c)			X			50.73(a)(2)(iv)			73.71(b)									
POWER LEVEL (10)			20.405(a)(1)(i)			50.38(c)(1)						50.73(a)(2)(v)			73.71(c)									
11010			20.405(a)(1)(ii)			50.38(c)(2)						50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
			20.405(a)(1)(iii)			50.73(a)(2)(i)						50.73(a)(2)(vii)(A)												
			20.405(a)(1)(iv)			50.73(a)(2)(ii)						50.73(a)(2)(vii)(B)												
			20.405(a)(1)(v)			50.73(a)(2)(iii)						50.73(a)(2)(ix)												

LICENSEE CONTACT FOR THIS LER (12)

NAME

L. R. Freeland, General Manager Nuclear Operations

TELEPHONE NUMBER

AREA CODE

4 1 2 6 4 3 - 1 2 5 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRCDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRCDS
X	W	I	X	X	X	X	X	X	N

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 2/5/93, with the Unit at 100 percent power, the steam generator blowdown (SGBD) system was being returned to service following the completion of a surveillance procedure on the SGBD radiation monitor. Local adjustments in SGBD flow were in progress to place the SGBD demineralizers in service. The SGBD tank level was within the normal operating level with the level control valve in automatic operation. The level control valve was then placed in manual and repositioned from 80% closed to 40% closed. At 1004 hours, the SGBD system isolated due to high level in the SGBD tank. When the isolation occurred the level indicator in the control room was showing the tank level to be approximately 4 inches below the isolation setpoint. This isolation is considered an Engineered Safety Features (ESF) component actuation by a non-ESF signal. The cause for the valve closure was an actual high level condition in the blowdown tank. This occurred due to discrepancies between the actual tank level and indicated tank level. It has been determined that the level transmitters, for the alarm and control functions, were not compensated for normal operating temperature and pressure. There were no safety implications as a result of this event. The steam generator blowdown isolation valves closed as designed on high level in the steam generator blowdown tank.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Beaver Valley Power Station Unit 2	0 5 0 0 0 4 1 2	9 3	0 0 3	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

On 2/5/93, with the Unit at 100 percent power, the steam generator blowdown (SGBD) system was being returned to service following the completion of a surveillance procedure on the SGBD radiation monitor. Local adjustments in SGBD flow were in progress to place the SGBD demineralizers in service. The SGBD tank level was at approximately 30 inches with the level control valve in automatic operation. The level control valve was then placed in manual and repositioned from 80% closed to 40% closed. At 1004 hours, the SGBD system automatically isolated due to high level in the SGBD tank. This isolation involves the automatic closing of valves 2BDG*AOV100A, 100B, and 100C, the steam generator outside containment isolation valves. When the blowdown isolation occurred the control room level indicator (2BDG-LI101A) was still reading approximately 30 inches. The isolation setpoint is set for a level of 34 inches and is controlled from a separate level switch (2BDG-LS105). This is a rigidly mounted Magnetrol Snap Switch with very little adjustment capability. It has been determined that the discrepancies between the actual tank level and indicated tank level are due to the fact that the level transmitters, for the alarm and control functions, were not compensated for normal operating temperature and pressure. This isolation is considered an Engineered Safety Features (ESF) component actuation caused by a non-ESF signal.

CAUSE OF THE EVENT

The cause of this event has been determined to be that the level transmitters, for the alarm and control functions, were not compensated for normal operating temperature and pressure. The calibration procedures are being revised to include steps that would compensate the level transmitters for normal operating temperature and pressure.

CORRECTIVE ACTIONS

The following corrective actions have been or will be taken:

1. The procedures used to calibrate the level transmitters are being revised to include steps that would compensate the level transmitters for normal operating temperature and pressure.
2. A Temporary Operating Procedure is being written to test the level transmitters and level switch to verify their proper operation.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20545, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Beaver Valley Power Station Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 1 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 3	0 0 3	0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

REPORTABILITY

This event was reported to the Nuclear Regulatory commission at 1058 hours on 2/05/93, in accordance with 10CFR50.72.b.2.ii, as an event involving an Engineered Safety Features component actuation by a non-ESF signal. This written report is being submitted in accordance with 10CFR50.73.a.2.iv, as an event involving an Engineered Safety Features component actuation.

SAFETY IMPLICATIONS

There were no safety implications as a result of this event. The steam generator blowdown isolation valves closed as designed on high level in the steam generator blowdown tank.

PREVIOUS OCCURRENCES

There have been two previously reported events involving steam generator blowdown system isolations due to a high level in the steam generator blowdown tank:

LER 92-013-00 ESF Actuation - Steam Generator Blowdown Isolation Due to Steam Generator Blowdown Level High caused by partially plugged tank discharge strainers,

LER 91-012-00 Steam Generator Blowdown System Isolation Due to High Level in the Blowdown Tank Due to Malfunctioning Temperature Switch.