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Refer to: RC-93-0055

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION
DOCKET NO. 50/395
OPERATING LICENSE NO. NPF-12
INTERIM PART 21 REPORT (SSH 930001, SSH 930002)

South Carolina Electric & Gas Company (SCE&G) submits this letter in accordance with 10CFR21.21(a)(2) as an interim report for two (2) nonconformances that have exceeded the sixty (60) day evaluation time limit.

On October 16, 1992, NCN 4542 was generated when anchor bolts of three (3) storage tanks (XTK-0039-MU, XTK-0025-RW and XTK-0008-CO) were found corroded by water and debris that had entered the anchor bolt sleeves. Design Engineering performed visual inspections of the anchor bolts and gathered data required for metal loss calculations to determine the adjusted stresses. It was concluded the anchor bolts are in need of repair but that the safety margin and allowable stresses are not significantly reduced, therefore allowing them to perform their designed function.

Currently the following repairs and analysis are being implemented by SCE&G:

1. Water and debris will be removed from the anchor bolt sleeves.
2. Verification of calculation for metal loss was conducted by Impell Corporation (Calculation number 0980-055-41-C-002) and compared to Gilbert/Commonwealth allowable tensile stresses (Calculation numbers: 2.05.4.2, 21.01 (CST), and 2.05.4.1 (RWST)).
3. Grout and RTV sealant will be applied, weather permitting, around bolts to meet the sleeves, thus preventing standing water around the bolt.
4. The design will be researched to determine if anchor bolts were improperly designed.
5. Determine 10CFR21 reportability.

SCE&G expects to complete the evaluation for NCN 4542 by July 16, 1993.

On November 3, 1992, NCN 4555 was initiated for the "A" train reactor building recirculation sump protection chamber drain valve (XVT0030A). The drain valve yoke nut was broken and therefore could not meet its design function to hold the valve closed. The recirculation sump protection chamber is self contained and does not communicate with the reactor building containment; therefore, a failure of the drain valve would not violate reactor building integrity.

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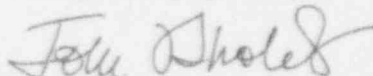
The valve body pressure boundary is maintained by a seal welded diaphragm; and because the valve outlet is sealed by a capped three-eighths (3/8) inch stainless steel tube, a failure of the yoke nut to hold the valve closed will not violate the recirculation sump protection chamber integrity. The valve and piping are seismically designed and will not fail during a seismic event.

SCE&G will evaluate the failure as follows:

1. The valve will be re-worked to return the component to its original design condition during RF 7.
2. Engineering Services will be notified of the rework, thus allowing an opportunity to collect "as found" information to initiate reportability evaluations.
3. An evaluation will be performed to determine the root cause of the yoke nut failure.

An appropriate evaluation cannot be completed until the valve rework has been accomplished and the root cause of the yoke nut failure determined. SCE&G expects to complete the 10CFR21 evaluation for reportability prior to July 16, 1993 upon completion of the valve rework and yoke nut failure determination.

Very truly yours,



John L. Skolds

JDG/JLS/nkk

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