

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

April 27, 1990

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 90-183
NAPS/PAK/TAH:bgp
Docket Nos. 50-338
50-339
License Nos. NPF-4
NPF-7

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNITS 1 AND 2
PROPOSED TECHNICAL SPECIFICATION CHANGE
RESIDUAL HEAT REMOVAL SYSTEM

Pursuant to 10 CFR 50.90, Virginia Electric and Power Company requests an amendment in the form of changes to the Technical Specifications (T.S.) of the Operating Licenses Nos. NPF-4 and NPF-7 for North Anna Power Station Units 1 and 2, respectively.

These changes will enhance Residual Heat Removal (RHR) system reliability by removing the automatic isolation requirement for the RHR Suction valves from Technical Specifications 3/4.7.9. A requirement to verify these valves closed and deenergized prior to exceeding 500 psig in the Reactor Coolant System (RCS) will be added.

Currently, Technical Specifications require an automatic closing function for the RHR suction valves whenever the RCS pressure exceeds 660 psig. This ensures double valve isolation between the high pressure RCS and the low pressure RHR system, but industry experience has shown it to be a major cause of inadvertent losses of decay heat removal capability. It has also been addressed specifically as being a major contributor to Loss of Decay Heat Removal events in Generic Letters 87-12 and 88-17. Deleting the interlock will increase the reliability of the RHR System during cold shutdown conditions while ensuring the isolation of the RHR System from the RCS will still be required by Technical Specifications.

The proposed change will also modify the surveillance testing requirements of the RHR pumps to be in accordance with Technical Specification 4.0.5 which invokes Section XI of the ASME Boiler and Pressure Vessel Code for inservice testing.

In addition, there are several administrative changes to specifications 3/4.7.9.1 and 3/4.7.9.2.

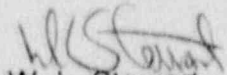
9005040062 900427
PDR ADCK 05000338
PDC

1/1
Accol Add:
NRB Bulukion, H. 1 1
NRB Trammell, C. 1 1
RES/DSIR/EIB 1 1
RES/DSR/PRAB 1 1

The proposed changes are provided in Attachments 1 and 2 for the current Technical Specifications and the MERITS Technical Specifications, respectively. The discussions, safety evaluations and the significant hazards determinations for these changes are enclosed in Attachment 3.

This request has been reviewed and approved by the Station Nuclear Safety and Operating Committee. It has been determined that the proposed changes do not involve an unreviewed safety question as defined in 10 CFR 50.59 or a significant hazards consideration as defined in 10 CFR 50.92.

Very truly yours,



W. L. Stewart

Senior Vice President - Nuclear

Attachments

- 1) Proposed Technical Specification Changes - STS Format
- 2) Proposed Technical Specification Changes - MERITS Format
- 3) Discussion, Safety Evaluation and Significant Hazards Determinations

cc: United States Nuclear Regulatory Commission
Region II
101 Marietta Street, N.W.
Suite 2900
Atlanta, GA 30323

Mr. M. S. Lesser
NRC Senior Resident Inspector
North Anna Power Station

Commissioner
Department of Health
Room 400
109 Governor Street
Richmond, Virginia 23219

COMMONWEALTH OF VIRGINIA)

COUNTY OF HENRICO)

The foregoing document was acknowledged before me, in and for the County and Commonwealth aforesaid, today by W. L. Stewart who is Senior Vice President - Nuclear, of Virginia Electric and Power Company. He is duly authorized to execute and file the foregoing document in behalf of that Company, and the statements in the document are true to the best of his knowledge and belief.

Acknowledged before me this 27 day of April, 1990.

My Commission Expires: May 31, 1994.

Vicki L. Hull
Notary Public

(SEAL)