



Arkansas Power & Light Company  
Arkansas Nuclear One  
Route 3, Box 137 G  
Russellville, AR 72801  
Tel 501 964 3100

April 16, 1990

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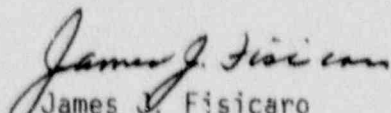
U. S. Nuclear Regulatory Commission  
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SUBJECT: Arkansas Nuclear One - Unit 1  
Docket No. 50-313  
License No. DPR-51  
Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 1 Monthly Operating Report for March, 1990  
is attached.

Very truly yours,

  
James J. Fisicaro  
Manager, Licensing

JJF/SAB/lw  
Attachment  
cc:

Mr. Robert D. Martin  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, TX 76011

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# OPERATING DATA REPORT

DOCKET NO: 50-313  
 DATE: March, 1990  
 COMPLETED BY: D. A. Schaubroeck  
 TELEPHONE: (501) 964-3743

## OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 1
2. Reporting Period: March 1-31, 1990
3. Licensed Thermal Power (Mwt): 2,568
4. Nameplate Rating (Gross MWe): 902.74
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 835
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
9. Power Level To Which Restricted. If Any (Net MWe): 80%
10. Reasons For Restrictions. If Any (Net MWe): A License Amendment was issued limiting operation to 80% due to a newly identified small break LOCA in the High Pressure Injection Line Piping.

	MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period ....	744.0	2,160.0	133,963.0
12. Number of Hours Reactor was Critical .....	669.2	2,085.2	93,296.4
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	5,044.0
14. Hours Generator On-Line .....	661.3	2,052.5	91,352.4
15. Unit Reserve Shutdown Hours ..	0.0	0.0	817.5
16. Gross Thermal Energy Generated (MWH) .....	1,348,804.0	4,151,003.0	206,175,458.0
17. Gross Electrical Energy Generated (MWH) .....	457,150.0	1,409,975.0	68,462,450.0
18. Net Electrical Energy Generated (MWH) .....	431,047.0	1,334,142.0	65,046,392.0
19. Unit Service Factor .....	88.9	95.0	68.2
20. Unit Availability Factor .....	88.9	95.0	68.8
21. Unit Capacity Factor (Using MDC Net) .....	69.3	73.9	58.1
22. Unit Capacity Factor (Using DER Net) .....	68.2	72.7	57.1
23. Unit Forced Outage Rate .....	11.1	5.0	13.7
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
<u>1R9 Refueling Outage is scheduled to begin September, 1990; and the scheduled date for restart is December, 1990.</u>			

25. If Shut Down At End of Report Period. Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-313  
UNIT: One  
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TELEPHONE: (501) 964-3743

MONTH March, 1990

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1	-34
2	-33
3	-34
4	221
5	656
6	656
7	658
8	660
9	660
10	655
11	660
12	661
13	661
14	660
15	660
16	660
17	660
18	660
19	661
20	662
21	662
22	662
23	662
24	665
25	663
26	663
27	663
28	662
29	663
30	663
31	663

AVGS: 579

## INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.



NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

MARCH, 1990

UNIT ONE

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Unit One began the month off line due to a leaking service water coil in a containment cooler.

On the fourth at 1044 hours, the unit was placed on line and power was escalated to 75% where it was held until power was stabilized. The unit attained 80% power on the fifth at 0030 hours and remained at that level through the end of the month. This limit is based on analysis pertaining to a postulated High Pressure Injection line break which allowed operation to that level only.

UNIT SHUTDOWNS AND POWER REDUCTIONS  
REPORT FOR MARCH, 1990

DOCKET NO	50-313
UNIT NAME	One
DATE	March, 1990
COMPLETED BY	D. A. Schaubroeck
TELEPHONE	(501)964-3743

<u>No.</u>	<u>Date</u>	<u>Type</u> <sup>1</sup>	<u>Duration</u> (Hours)	<u>Reason</u> <sup>2</sup>	<u>Method of</u> <u>Shutting</u> <u>Down Reactor</u> <sup>3</sup>	<u>Licensee</u> <u>Event</u> <u>Report #</u>	<u>System</u> <u>Code</u> <sup>4</sup>	<u>Component</u> <u>Code</u> <sup>5</sup>	<u>Cause &amp; Corrective</u> <u>Action To</u> <u>Prevent Recurrence</u>
90-03	900301	F	82.7	A	4	1-90-001	BI	CCL	Unit off line for work on a leaking coil in a containment cooler.

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training &  
License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation  
5-Load Reduction  
9-Other

<sup>4</sup>  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
1022)

<sup>5</sup>  
Exhibit I - Same Source

DATE: March, 1990

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 1
2. Scheduled date for next refueling shutdown. September, 1990
3. Scheduled date for restart following refueling. December, 1990
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Normal Technical Specification changes associated with submission of the ANO-1 Cycle 10 Reload Report.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. July, 1990
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

A debris resistant, extended solid end cap design fuel rod will be used in the reload fuel batch.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 508
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  
present 968 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1994 (Loss of fullcore offload capability)