



Arkansas Power & Light Company  
Arkansas Nuclear One  
Route 3, Box 137 G  
Russellville, AR 72801  
Tel 501 964 3100

April 16, 1990

2CAN049015

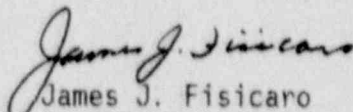
U. S. Nuclear Regulatory Commission  
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SUBJECT: Arkansas Nuclear One - Unit 2  
Docket No. 50-368  
License No. NPF-6  
Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 2 Monthly Operating Report for March, 1990 is attached.

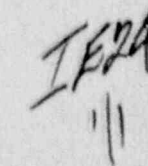
Very truly yours,

  
James J. Fisicaro  
Manager, Licensing

JJF/SAB/lw  
Attachment  
cc:

Mr. Robert D. Martin  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, TX 76011

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# OPERATING DATA REPORT

DOCKET NO: 50-368  
 DATE: March, 1990  
 COMPLETED BY: M. S. Whitt  
 TELEPHONE: (501) 964-3743

## OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: March 1-31, 1990
3. Licensed Thermal Power (Mwt): 2,815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: \_\_\_\_\_
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

	MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period ....	744.0	2,160.0	87,792.0
12. Number of Hours Reactor Was Critical .....	669.5	1,944.2	64,308.4
13. Reactor Reserve Shutdown Hours .....	0.0	0.0	1,430.1
14. Hours Generator On-Line .....	665.0	1,936.6	62,717.3
15. Unit Reserve Shutdown Hours ..	0.0	0.0	75.0
16. Gross Thermal Energy Generated (MWH) .....	1,735,876.0	5,116,612.0	163,325,142.0
17. Gross Electrical Energy Generated (MWH) .....	574,885.0	1,693,205.0	53,650,681.0
18. Net Electrical Energy Generated (MWH) .....	546,441.0	1,612,132.0	50,989,102.0
19. Unit Service Factor .....	89.4	89.7	71.4
20. Unit Availability Factor .....	89.4	89.7	71.5
21. Unit Capacity Factor (Using MDC Net) .....	85.5	87.0	67.7
22. Unit Capacity Factor (Using DER Net) .....	80.5	81.8	63.7
23. Unit Forced Outage Rate .....	10.6	10.3	13.7
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			
25. If Shut Down At End of Report Period. Estimated Date of Startup: _____			
26. Units in Test Status (Prior to Commercial Operation): _____			

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368  
 UNIT: Two  
 DATE: March, 1990  
 COMPLETED BY: M. S. Whitt  
 TELEPHONE: (501) 964-3743

MONTH March, 1990

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-Net)

1 .....	888
2 .....	889
3 .....	890
4 .....	359
5 .....	-25
6 .....	-24
7 .....	6
8 .....	588
9 .....	626
10 .....	621
11 .....	604
12 .....	603
13 .....	707
14 .....	880
15 .....	890
16 .....	890
17 .....	891
18 .....	892
19 .....	891
20 .....	888
21 .....	892
22 .....	891
23 .....	893
24 .....	895
25 .....	895
26 .....	894
27 .....	893
28 .....	892
29 .....	890
30 .....	891
31 .....	891

AVGS: 735

## INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

# NRC MONTHLY OPERATING REPORT

## OPERATING SUMMARY

MARCH 1990

UNIT TWO

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The unit began the month operating at 100% full power (FP).

On the fourth at 0812 hours, a power descent was commenced due to noise in the "D" Core Protection Calculation (CPC) channel. At 1144 hours the unit was taken off line to trouble shoot the source of the noise. The noise source was determined to be caused by a faulty surge capacitor on a Reactor Coolant Pump (RCP). Following replacement of surge capacitors on all four RCPs, the unit was placed back on line on the seventh at 1842 hours. The unit obtained a power level of 70% on the eighth at 0324 hours, where it was held due to a lack of demand on the system. During the power hold a condenser tube leak was suspected due to elevated sodium concentrations in the Condenser/Feedwater System. Following the search for the source of the sodium in-leakage and the release from the power hold by the dispatcher, the unit recommenced a power escalation on the thirteenth at 1034 hours.

On the fourteenth at 0355 hours, the unit attained 100% FP and remained at that level through the end of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS  
REPORT FOR MARCH, 1990

DOCKET NO.	50-368
UNIT NAME	Two
DATE	March, 1990
COMPLETED BY	M. S. Whitt
TELEPHONE	501-964-3743

<u>No.</u>	<u>Date</u>	<u>Type<sup>1</sup></u>	<u>Duration (Hours)</u>	<u>Reason<sup>2</sup></u>	<u>Method of Shutting Down Reactor<sup>3</sup></u>	<u>Licensee Event Report #</u>	<u>System Code<sup>4</sup></u>	<u>Component Code<sup>5</sup></u>	<u>Cause &amp; Corrective Action to Prevent Recurrence</u>
90-01	900304	F	79	A	1	2-90-005	AB	CAP	The unit was taken off line due to noise on the "D" CPC channel. The source of the noise was determined to be a faulty surge capacitor on a RCP. Before the unit was placed back on line, the surge capacitors on all four RCPs were replaced.

1  
F: Forced  
S: Scheduled

2  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training &  
License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation  
5-Load Reduction  
9-Other

4  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
1022)  
5  
Exhibit I - Same Source

DATE: March, 1990

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. February 1991  
(Beginning of Cycle 8 criticality was 11/18/89)

3. Scheduled date for restart following refueling. April, 1991

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

None Expected. Reload fuel design is in progress.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. None Required

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

To obtain the presently planned cycle 8 length of 420 EFPD, it will be necessary to raise the current peak rod burnup limits. A report justifying an increase was submitted in July, 1989.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 421

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

present 988 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1996 (Loss of fullcore offload capability)