



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
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April 5, 1990

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for March, 1990.

Very truly yours,

WR
for G. J. Diederich
Station Manager
LaSalle County Station

GJD/JWT/crh

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761, extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical generator on-line at 3320 MWT.
5	2400	Reduced load to 3000 MWT for CRD exercising.
6	1300	Returned to full power.
12	0200 1000	Reduced load to 3090 MWT for System load. Returned to full power.
13	0030 0700	Reduced load to 3190 for CRD exercising. Returned to full power.
14	0200 1000	Reduced load to 2970 MWT for System load. Returned to full power.
15	0200 1000	Reduced load to 2530 MWT for System load. Returned to full power.
20	0030 1000	Reduced load to 3174 MWT for CRD exercising. Returned to full power.
21	0400 1200	Reduced load to 2500 MWT for Surveillances and control-rod moves. Returned to full power.
22	0000 1000	Reduced load to 3000 MWT for System load. Returned to full power.
24	1230 1600	Reduced load to 3181 MWT for cooling problems on 1W Main transformer. Returned to full power.
27	0000	Reduced power to 3125 MWT for CRD exercising.
28	0340	Reactor scram, generator trip, due to failure of an insulator on 1W Main transformer.
30	0030	Reactor critical.
31	0310	Generator on-line.
	2400	Reactor critical, generator on-line at 2216 MWT.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to the Facility License or Technical Specification.
 - a. Clarified DG Start requirements.
 - b. Clarified DG fast-start/load requirements.
 - c. Revised DG test schedule per Generic Letter.
2. Changes to procedures which are described in the Safety Analysis Report.
 - a. LaSalle Special Procedure LLP-90-014, "Operating Procedure Votes 100 System". This procedure provides instruction for the periodic testing of valve operators using the Valve Operation Test and Evaluation System (VOTES).
 - b. LaSalle Operating Procedure LOP-RH-07, "Shutdown Cooling System Startup and Operation."
Addition to Limitation and Actions to allow the use of 1(2)E12-F065A/B RHR Heat Exchanger Steam Condensing Outlet and 1(2)E12-F011A/B, RHR Heat Exchanger Steam Condensing to Supp. Chamber Stop Valves for Reactor Vessel Level Control/Water inventory.
3. Tests and Experiments not described in the Safety Analysis Report.
(None)
4. Major corrective maintenance to Safety-Related Equipment, including any SOR switch failure reports.
(See Table 1)
5. Completed Safety-Related Modifications.
(See Table 2)

B.4 TABLE 1

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
(Unit 0)				
None				
(Unit 1)				
L97999	Personnel air-lock outer door upper shaft seal.	Worn excessively	Leakage past seal	Replaced seal

(No SOR Switch Failures)

COMPLETED SAFETY-RELATED MODIFICATIONS

<u>NUMBER</u>	<u>DESCRIPTION</u>
M01-0-88-002	Relocate the Reboiler Leak Detection Radiation Monitor 1(2)D18-K752 to the far right of the radiation monitor grouping for the Control Room HVAC Intake Radiation Monitors. This is performed in accordance with Detailed Control Room Design Review (DCRDR).
M01-0-84-111	Installation of the HACR-IV synchronizing check relay in place of the existing Westinghouse CVE synchronizing check relay and its associated wiring.
M01-1-86-083	Replace existing Reactor Water Clean Up (RWCU) Lonegran relief valves 1G33-F340A and 1G33-F340B with slow-opening valves.
M01-1-87-027	Relocate the oxygen breathing lines which have created a tripping hazard for control room personnel. This is performed in accordance with Detailed Control Room Design Review (DCRDR).

C. LICENSEE EVENT REPORTS

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
90-005-00	03/27/90	Fire Detection Zone OOS > 14 Days and no special report submitted.
90-006-00	03/28/90	Reactor Scram Due to Loss of Main Transformer.

D. DATA TABULATIONS

1. Operating Data Report (attached)
2. Average Daily Unit Power Level (attached)
3. Unit Shutdowns and Power Reductions (attached)

D.1 OPERATING DATA REPORT

DOCKET NO. 050-273
 UNIT LASALLE ONE
 DATE April 10, 1990
 COMPLETED BY J.W. THUNSTEDT
 TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: MARCH 1990 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEMAND CAPACITY (MWe-Net): 1,036
 DESIGN ELECTRICAL RATING (MWe-Net) 1,078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (None)
 4. REASONS FOR RESTRICTION (IF ANY): (N/A)

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	699.1	1,989.0	34,123.1
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,641.2
7. TIME GENERATOR ON-LINE (HOURS)	672.5	1,862.1	33,280.4
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MMWt-Gross)	2,172,000	5,726,279.4	93,559,847.4
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	740,756	1,951,380	31,019,921
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	715,591	1,884,422	29,617,124
12. REACTOR SERVICE FACTOR (%)	94.0	92.1	62.3
13. REACTOR AVAILABILITY FACTOR (%)	94.0	92.1	65.3
14. SERVICE FACTOR (%)	90.4	86.2	60.7
15. AVAILABILITY FACTOR (%)	90.4	86.2	60.7
16. CAPACITY FACTOR (USING MDC) (%)	92.8	84.2	52.2
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	89.2	80.9	50.1
18. FORCED OUTAGE FACTOR (%)	9.6	6.6	9.9

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 (None)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 (N/A)

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE April 10, 1990
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

REPORT PERIOD: MARCH 1990

DAY	POWER	DAY	POWER
1	1,099	17	1,094
2	1,100	18	1,097
3	1,099	19	1,099
4	1,100	20	1,088
5	1,098	21	1,023
6	1,050	22	1,060
7	1,100	23	1,105
8	1,103	24	1,098
9	1,102	25	1,093
10	1,101	26	1,095
11	1,098	27	1,088
12	1,082	28	155
13	1,087	29	-15
14	1,080	30	-16
15	1,050	31	305
16	1,091		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%

YEARLY SEQUENTIAL NUMBER	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
11	15	S	0	F	5	System load
12	21	S	0	B	5	Surveillances
13	28	F	71.5	A	3	Phase B-to-ground fault on 1W Main Transformer

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT
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(None)

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE
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(Note: the year and unit numbers have been omitted from the Outage Number)

(Unit 0)

(None)

(Unit 1)

139	RCIC	(Administrative)
140	RCIC	Inspect brushes
141	RCIC	Replace oil on Water-Leg Pump
148	1E12-C300C(RH)	Lubricate coupling.
152	1DG01K	Replace air-start motor
155	1E12-C300A(RH)	Lubricate coupling
156	1E12-C200B(RH)	Lubricate coupling
157	1B D/G	Replace oil
181	1DG035 (LP)	(Administrative)

3. Changes to the Off-Site Dose Calculation Manual
(None).

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MARCH, 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 650-374

LICENSE NO. NPF-18

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2. Changes to procedures which are described in the Safety Analysis Report.
3. Tests and Experiments not covered in the Safety Analysis Report.
4. Corrective Maintenance of Safety-Related Equipment
5. Completed Safety Related Modifications

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
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E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations
2. ECCS System Outages
3. Off-Site Dose Calculation Manual Changes
4. Major Changes to Radioactive Waste Treatment System
5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761 extension 2453.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE

Day	Time	Event
1	0000	Reactor critical, generator on-line at 3275 MWT.
6	2400	Reduced load to 3000 MWT for CRD exercising.
7	1300	Returned to full power.
13	0300	Reduced load to 2690 MWT for Motor-driver Reactor feed-pump test.
	1300	Returned to full power.
	2300	Reduced power to 2910 MWT for CRD exercising.
14	1600	Returned to full power.
17	0028	Generator off-line, Refueling outage (L2R03) began.
	0118	Reactor manually shut-down (subcritical)
31	2400	Reactor subcritical, generator off-line, L2R03 in progress.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to the Facility License or Technical Specification.
 - a. Clarified DG Start requirements.
 - b. Clarified DG fast-start/load requirements.
 - c. Revised DG test schedule per Generic Letter.
2. Changes to procedures which are described in the Safety Analysis Report.
 - a. LaSalle Special Procedure LLP-90-014, "Operating Procedure Votes 100 System". This procedure provides instruction for the periodic testing of valve operators using the Valve Operation Test and Evaluation System (VOTES).
 - b. LaSalle Operating Procedure LOP-RH-07, "Shutdown Cooling System Startup and Operation."
Addition to Limitation and Actions to allow the use of
1(2)E12-F065A/B RHR Heat Exchanger Steam Condensing Outlet and
1(2)E12-F011A/B, RHR Heat Exchanger Steam Condensing to Supp.
Chamber Stop Valves for Reactor Vessel Level Control/Water
inventory.
3. Tests and Experiments not described in the Safety Analysis Report.
(None)
4. Major corrective maintenance to Safety-Related Equipment, including any SOR switch failures.
(See attached report for SOR 2E31-N011D)
5. Completed Safety-Related Modifications.
(None)

SOR dp Switch Failure Data Sheet

Equipment Piece Number: 2E31-N011D Model Number: 102AS-B305-NX-JJTTX6

Serial Number: 86-8-1916

Application: Main Steam Line High Flow Isolation Switch

Date and Time of Discovery: 03/27/90 0500 hours

Reactor Mode: 5 (Refuel) Power Level: 0%

Calibration Tolerance: 102.0 - 104.0 PSID

Nominal Setpoint: 103.0 PSID

Action Limits: <98.2 or >107.8 PSID

Reject Limits: <94.8 or >111.2 PSID

Technical Specification

Limit: 116.0 PSID

As Found Setpoint: 87.0 PSID

Date and Time of Return to Service: 03/27/90 0610 hours

Model Number of Replacement Switch: 102AS-B305-NX-JJTTX6

Serial Number of Replacement Switch: 86-10-559

Cause: Unknown at this time. Suspect instrument setpoint drift.

Corrective Action: Replaced switch. In process of inspecting failed switch.

DVR Number: 1-2-90-024

C. LICENSEE EVENT REPORTS

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
90-003-00	03/17/90	Inboard Shutdown Cooling SDC Isolation during initiation of Shutdown Cooling
90-004-00	03/20/90	LLRT Minimum Path Failure of >0.6LA

D. DATA TABULATIONS

1. Operating Data Report (Attached)
2. Average Daily Unit Power Level (Attached)
3. Unit Shutdowns and Power Reductions (Attached)

D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LASALLE TWO
 DATE April 10, 1990
 COMPLETED BY J.W. THUNSTEDT
 TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: MARCH 1990 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
 DESIGN ELECTRICAL RATING (MWe-Net) 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 0

4. REASONS FOR RESTRICTION (IF ANY): Refueling (L2R03)

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	305.3	1,764.1	31,911.2
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,716.9
7. TIME GENERATOR ON-LINE (HOURS)	384.5	1,748.7	31,431.8
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh-Net)	1,185,984	5,586,967.2	91,907,015.2
10. ELECTRICAL ENERGY GENERATED (MWe-Net)	406,280	1,907,042	30,358,768
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	388,613	1,837,998	29,073,303
12. REACTOR SERVICE FACTOR (%)	51.8	81.7	66.8
13. REACTOR AVAILABILITY FACTOR (%)	51.8	81.7	70.4
14. SERVICE FACTOR (%)	51.7	81.0	65.8
15. AVAILABILITY FACTOR (%)	51.7	81.0	65.8
16. CAPACITY FACTOR (USING MDC) (%)	50.4	82.1	58.7
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	48.5	78.9	56.4
18. FORCED OUTAGE FACTOR (%)	0.0	2.9	15.7

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 Refueling, (In PROGRESS), 12 Weeks

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 JUNE 6, 1990

DOCKET NO. 050-374

UNIT LASALLE TWO

DATE April 10, 1990

COMPLETED BY J.W. THUNSTEDT

TELEPHONE (815)-357-6761

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

REPORT PERIOD: MARCH 1990

DAY	POWER	DAY	POWER
1	1,077	17	-11
2	1,078	18	-12
3	1,074	19	-12
4	1,070	20	-12
5	1,065	21	-12
6	1,061	22	-12
7	1,041	23	-12
8	1,061	24	-12
9	1,054	25	-12
10	1,050	26	-12
11	1,047	27	-12
12	1,041	28	-14
13	1,008	29	-15
14	1,013	30	-16
15	1,034	31	-12
16	606		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%

YEARLY SEQUENTIAL NUMBER	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON CODE	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
6	17	S	0	C	5	L2R03

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None)

2. ECCS System Outages

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
259	LPCS flushing Water Spool Piece	Install Spool
268	2B D/G	Repair 2D0014, 2E22-F362B
270	HPCS	Install flush line from CY
271	2VY03C	Clean Cooling coils
272	HPCS	Perform LES-HP-202 and 203
276	2E12-C003 (RH)	Inspect and clean 2E12-F084.
277	2B D/G	Calibrate relays.
278	2VY02C	Clean cooling coils.
285	2E12-F042B(RH)	Perform EQ inspection on breaker.
287	2B D/G	Perform calibrations.
295	HPCS	Repair 2E22-F007
300	2B D/G	Perform LES-DG-202
404	HPCS	Repair Water-leg pump
405	HPCS	Meggar motor
406	2E22-F012(DG)	Replace contactors
423	LPCS	Replace bearing oil
424	RHR	Perform LES-GM-103
425	2E12-C300D (RH)	Replace contactor springs
431	2E12-C001C (RH)	Perform LES-GM-103
433	LPCS	Replace motor bearing oil

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
480	2E12-F016B (RH)	Perform EQ Inspection or breakers
484	2E22-F319 (DG)	Perform EQ inspection
488	2E12-C002C (RH)	Perform EQ inspection
529	2E51-F086 (RI)	Perform MOV actuator inspection
531	2E51-F080 (RI)	Perform MOV actuator inspection
536	RCIC	Inspect relays
552	HP	Relocate instruments
706	RH	Perform LOP-RH-07
709	RH	Perform LOP-RH-07
711	2E12-F004B (RH)	Repair operator
733	Div III logic (HP)	Replace level switches
734	HP	Replace cable and switches
767	RH	(Various)
768	RH	(Various)
784	HPCS	Fill and Vent system
787	2E12-F063B (RH)	Fill Suppression Pool
793	RH	Perform modification on 2PM16J
807	RCIC	(Administrative)
811	2E12-C003D (RH)	Inspect and repair discharge check valve
812	2E12-C300C (RH)	Inspect and repair discharge check valve
813	Div II (RH)	Inspect relays
817	RH	(Administrative)
831	2E22-C001 (HP)	Perform LES-GM-106
840	2E12-F009 (RH)	Perform LES-AP-201
841	2E12-C002B (RH)	Megger motor
842	2E12-F336B (RH)	Perform EQ inspection

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
844	Div II (DG)	Perform LES-RH-201
851	2E12-F003B (RH)	Repair 2VR05YA
852	2E12-F068B (RH)	Repack valve
881	2B RHR Pump	Remove thermocouple
882	2C RHR Pump	Remove thermocouple
883	2B RHR Pump	(Administrative)
885	2E12-F016A (RH) 2E12-F017A (RH) 2E51-F080 (RH)	Perform LIS-RH-14
888	2B RHR Pump	Remove thermocouples
889	2C RHR Pump	Remove thermocouples
891	LPCS Pump	Remove extra wiring from C/S
892	2E12-F024B (RH)	Repack valve
896	2DG01K	(Administrative)
3.	Changes to the Off-Site Dose Calculation Manual (None).	
4.	Major changes to Radioactive Waste Treatment Systems. (None)	
5.	Indications of Failed Fuel Elements. (None)	