



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

April 17, 1990

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

Dear Sir:

Licensee Event Report #90-004-00, Docket #050-374 is being
submitted to your office in accordance with
10CFR50.73(a)(2)(ii).

G. J. Diederich
Station Manager
LaSalle County Station

GJD/JPP/kg

Enclosure

xc: Nuclear Licensing Administrator
NRC Resident Inspector
NRC Region III Administrator
INPO - Records Center

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LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) LoSalle County Station Unit 2 Docket Number (2) 0 | 5 | 0 | 0 | 0 | 3 | 7 | 4 Page (3) 1 of 0 | 4

Title (4)

Local Leak Rate Test Minimum Pathway Leakage of Greater than 0.6 La Limits

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Name	Docket Number(s)
0 3	2 0	9 0	9 0	0 0 4	0 0	0 4	1 7	9 0		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9) 5 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	Other (Specify in Abstract below and in Text)
20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name James P. Peters, Technical Staff Engineer, extension 2325 TELEPHONE NUMBER AREA CODE 8 | 1 | 5 3 | 5 | 7 | - | 6 | 7 | 6 | 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPDOS
X	C	E	I S V A 3 9 1	Y	X	S	J	I S V A 3 9 1	Y
X	H	K	I S V	Y	X	V	B	I S V A 3 9 1	Y

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15) X | Yes (If yes, complete EXPECTED SUBMISSION DATE) | NO | Month | Day | Year 0 | 7 | 3 | 1 | 9 | 0

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 20, 1990, while Unit 2 was in Operational Condition 5 (Refuel), for its third refueling outage, local leak rate tests had been performed on 2G33-F001 and 2G33-F004, Reactor Water Cleanup (RWC) Suction Isolation Valves, and various other Primary Containment valves. These tests resulted in a minimum pathway leakage which exceeded 0.6 La limits. The exact cause of this occurrence is unknown at this time. All failures will be repaired and satisfactorily retested prior to startup from the current refueling outage. This report will be supplemented with the causes and corrective actions.

This event is being reported pursuant to the requirements of 10CFR50.73(a)(2)(ii).

LICENSEE EVENT REPORT (LER) FAILURE CONTINUATION														
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						Page (3)						
		Year		Sequential Number		Revision Number								
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	9	0	-	0 0 4	-	0 0	0 2	OF	0 4				

LaSalle County Station Unit 2	0	5	0	0	0	3	7	4	9	0	-	0	0	4	-	0	0	0	2	OF	0	4
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION												Form Rev 2.0	
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						Revision Number	Page (3)				
		Year	///	Sequential Number	///								
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	9 0	-	0 0 4	-			0 0	0 3	OF	0 4		

TEXT Energy Industry Identification System (EIIIS) codes are identified in the text as [XX]

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIIS) codes are identified in the text as [XX].

A. CONDITION PRIOR TO EVENT

Unit(s): 2 Event Date: 3/20/90 Event Time: 0800 Hours
 Reactor Mode(s): 5 Mode(s) Name: Refuel Power Level(s): 0%

B. DESCRIPTION OF EVENT

On March 20, 1990, while Unit 2 was in Operational Condition 5 (Refuel), for its third refueling outage, LaSalle Technical Surveillance LTS-100-19, "Reactor Water Cleanup Suction Local Leak Rate Test," had been performed on the 2G33-F001 and 2G33-F004 (RT) [CE] valves. These valves are the Reactor Water Cleanup Suction Containment Isolation valves. A minimum-pathway leakage of greater than 231.4 SCFH was observed which exceeded the 0.6 La limits per 10CFR50 Appendix J and Technical Specification 3.6.1.2, Primary Containment Integrity (PC) [NH]. Additional 10CFR50 Appendix J local leak rate testing failures have occurred to date during this outage.

This event is being reported pursuant to the requirements of 10CFR50.73(a)(2)ii).

C. APPARENT CAUSE OF EVENT

The exact cause of these failures is unknown at this time. Maintenance activity will be completed to reduce the maximum pathway leakage to less than 0.6 La prior to startup. An inspection of these valves will be made prior to and/or during repairs such that a mode of failure can be determined. A supplemental report will be submitted to document the apparent cause of failure. The supplemental report will also include other local leak rate failures which occurred during the outage.

D. SAFETY ANALYSIS OF EVENT

Exceeding this limit did not by itself pose any significant risks or hazards to the public since the total leakage determined by Type B and C tests does not definitively represent the probable leakage from the Containment under accident conditions.

The worst case (maximum-pathway) total leakage path calculation is still not a true measure of expected leakage during accident conditions. For example, a number of Type C tests are performed on systems which would, under most accident scenarios, be filled with water and pressurized. These valves, while they may represent a substantial portion of the total measured leakage for Type B and C testing, would contribute little or nothing to a radiological release under most accident conditions.

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TEXT Energy Industry Identification System (EIIIS) codes are identified in the text as [XX]												

E. CORRECTIVE ACTIONS

Work Requests were generated to repair the valves. A local leak rate test will be performed at the completion of repair work to ensure containment integrity. Action Item Record #374-200-90-02101 was generated to document and track the cause of failure through a Supplemental Licensee Event Report for these valves and other local leak rate failures occurring during the outage.

F. PREVIOUS EVENTS

LER Number	Title
83-107/31-0	Numerous Containment Isolation Valves Exceeding Allowable Leakage
83-146/03X-1	Inboard Feedwater Check Valves Failed Local Leak Rate Tests
373/84-012-00	Feedwater Checks and Drywell Equipment Drains Failed Local Leak Rate Tests
373/84-064-01	1B21-F010A/32A and 1E51-F064 Failed Local Leak Rate Tests
373/85-066-01	Containment Isolation Valve Failed Local Leak Rate Test
374/87-002-01	Containment Leakage Limit Exceeded
373/88-002-01	Type B and C Total Leakage Exceeded 0.6 La During Local Leak Rate Testing
374/88-014-01	Type B and C Total Leakage Exceeded 0.6 La During Local Leak Rate Testing

G. COMPONENT FAILURE DATA

Manufacturer	Nomenclature	Model Number	MFG Part Number
Anchor Darling	2G33-F001/4 (MOV) Gate Valve	94-13753(4)	
Anchor Darling	2B21-F010A Testable Check Valve	3600-3	
WKM Div/ACF Ind. Inc.	2RF012/13 (AOV) Gate Valve	70-29-1	
Anchor Darling	2VP053A (MOV) Gate Valve	93-14412	
Anchor Darling	2HG006A (MOV) Gate Valve	93-14409	
Rockwell Mfg. Co.	2E12-F053A (MOV) Globe Valve	4016MTY	