WOLF CREEK
NUCLEAR OPERATING CORPORATION

Bart D. Withers
President and
Chief Executive Officer

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WM 90-0076

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, D. C. 20555

Reference: Letter dated January 9, 1990 from D. V. Pickett, NRC to B. D. Withers, WCNOC
Subject: Docket No. 50-482: Response to Request for Additional Information Concerning the Draft Risk-Based Inspection Guide for Wolf Creek Generating Station

Gentlemen:

The purpose of this letter is to respond to the request for additional information concerning the draft Risk-Based Inspection Guide for Wolf Creek Generating Station (WCGS). Wolf Creek Nuclear Operating Corporation volunteered to participate in the Risk-Based Inspection Guide program and in March 1989 provided specific plant documents to the NRC staff's contractor for use in developing a plant-specific inspection guide.

The Reference provided the draft Risk-Based Inspection Guide for WCGS and requested additional information for completing the guide. The attachment to this letter provides the requested information.

If you have any questions concerning this matter, please contact me or Mr. H. K. Chernoff of my staff.

Very truly yours,

BDWithers

Bart D. Withers
President and
Chief Executive Officer

BDW/jad

Attachment

cc: R. D. Martin (NRC), w/a
D. Persinko (NRC), w/a
D. V. Pickett (NRC), w/a
M. E. Skow (NRC), w/a

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Response to Request for Additional Information Concerning the Draft Risk-Based Inspection Guide for Wolf Creek Generating Station

Question:

1. Whether the air supply to the pressurizer PORVs is backed by nitrogen bottles or air accumulators.

Response:

The Pressurizer Power Operated Relief Valves (PORVs), BB-PCV-455A and BB-PCV-456A, are solenoid actuated valves which respond to a signal from a pressure sensing system or to manual control.

Question:

2. The number of PORV actuations that the backup supply is expected to support.

Response:

Not applicable to Wolf Creek Generating Station (WCGS). See response to Question 1.

Question:

3. The number of PORVs that would be required to open to achieve sufficient core cooling in the "feed and bleed" mode.

Response:

Procedure EMG FR-H1, "Response to Loss of Secondary Heat Sink" requires two PORVs to be opened to achieve sufficient core cooling in the "feed and bleed" mode.

Question:

4. The success criteria for the PORVs/SRVs during ATWS events.

Response:

In response to 10 CFR 50.62 "Requirements for Reduction of Risk from Anticipated Transients Without Scram (ATWS) Events for Light-Water-Cooled Nuclear Power Plants", Westinghouse performed a generic analysis which indicated a success criteria of 3 safety relief valves or 2 safety relief valves and 2 PORVs for pressure relief during ATWS events. As part of the Independent Plant Examination required by Generic Letter 88-20, WCNOC will utilize this success criteria.

Question:

5. The specific locations (e.g., containment elevations) of the SRVs, PORVs and block valves.

Response:

The PORVs, BB-PCV-455A and BB-PCV-456A, are located at elevation 2070 ft. in the containment building. The pressurizer safety relief valves, BB-V-8010A, BB-V-8010B, BB-V-8010C, are at an elevation of approximately 2080 ft. The PORV block valves BB-HV-8000A and BB-HV-8000B are at elevation 2070 ft., adjacent to BB-PCV-455A and BB-PCV-456A.

Question:

6. The PORV control power supply breaker designations and locations.

Response:

The control power supply breaker designations for BB-PCV-455A and BB-PCV-456A are NK5018 and NK4421, respectively. The supply breakers are located internal to control panels BB07 and BB08 on elevation 2016 ft.

Question:

7. The ID number of the discharge check valves for the train B ESW pump.

Response:

The train B Essential Service Water pump discharge check valve is EF-V-004.

Question:

8. The procedure number for periodic testing of the turbine-driven AFW pump.

Response:

Procedure STS AL-103, "Turbine Driven Aux FW Pump Inservice Pump Test" is utilized to demonstrate operability as required by Technical Specification 4.0.5 and 4.7.1.2.1.a.2.