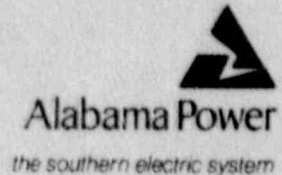


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W. G. Hairston, III
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10CFR50.55a(g)

April 12, 1990

Docket No. 50-364

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Joseph M. Farley Nuclear Plant - Unit 2
Response to the NRC Request for Additional Information
for the Inservice Inspection Program

By letter dated August 3, 1989, the NRC requested that Alabama Power Company provide additional information pertaining to the review of the Unit 2 updated Inservice Inspection (ISI) Program. Revision 0 of this program was submitted to the NRC by letter dated December 16, 1988. The NRC granted interim approval of certain reliefs by letter dated March 31, 1989. As discussed in letters dated February 8 and March 23, 1989, approval of the remaining relief requests was deferred. Alabama Power Company proposed a schedule for approval of the remaining reliefs by letter dated June 8, 1989.

Alabama Power Company responded to the NRC's Request for Additional Information (RAI), by letters dated October 5 and December 7, 1989. In a conference call held January 30, 1990, the NRC Staff requested answers to several questions resulting from review of these submittals. These questions were answered in a subsequent conference call on March 7, 1990. As agreed in the conference calls, enclosed is a summary of the NRC's questions and Alabama Power Company's responses.

It is requested that the NRC notify Alabama Power Company upon completion of its review of the RAI response and inform Alabama Power Company as to the disposition of each item. Upon receipt of confirmation that all issues have been satisfactorily resolved, Alabama Power Company will submit

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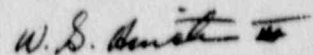
Revision 1 of the ISI Program which will incorporate the changes discussed in the RAI responses.

It is also requested that the NRC's review of the ISI Program be completed and SER issued in accordance with the scheduled provisions of the June 8, 1989 letter referenced above. Resolution of the open issues on the remaining relief requests and their granting in accordance with the proposed schedule will provide approximately one year (not including the Code allowed one-year extension) for Alabama Power Company to complete the required examinations. This will also permit proper planning of examinations to be performed during the remaining refueling outage tentatively scheduled to begin September 14, 1990.

If there are any questions or if additional information is needed, please advise.

Respectfully submitted,

ALABAMA POWER COMPANY



W. G. Hairston, III

WGH/DEM/STB:cht-nrc.8.24

Enclosure

cc: Mr. S. D. Ebnetter
Mr. S. T. Hoffman
Mr. G. F. Maxwell
Mr. B. Brown

ENCLOSURE

Joseph M. Farley Nuclear Plant - Unit 2
Response to the NRC's Request for Additional
Information for the Inservice Inspection Program

The following questions and answers pertaining to the subject RAI were discussed in conference calls held between Alabama Power Company and the NRC staff:

1. Piping Design and Operating Pressures
(Reference NRC RAI, Enclosure 2, Item 2, Section M)

NRC Question:

Explain why Attachment 7 to APCo's October 5, 1989 reply lists several lines with operating pressures which exceed the design pressures.

APCo Reply:

A design review of the lines in question was performed and the existing design and operating pressures and temperatures were evaluated against applicable ASME Code, Section III requirements. This evaluation concluded that the piping as designed is acceptable for the design and operating conditions specified. Limiting design pressures for the maximum design temperature were calculated for each of the lines and in all cases the limiting design pressures greatly exceeded the operating and design pressures originally specified. Furthermore, piping designed for 2485 psig at 650°F is acceptable for service at higher operating pressures (than 2485 psig) and correspondingly lower temperatures (than 650°F) provided the ratios of pressure to allowable stress at a given temperature are similar. The evaluation confirmed that in each case where the operating pressure exceeded the design pressure, the operating temperature is less than the design temperature such that the operating pressure to allowable stress ratio is equivalent to or more conservative than the design pressure to allowable stress ratio.

As a result of the review a determination was made that more realistic operating pressures should be specified and the new values are provided in the revised Attachment 7 (copy attached). In some cases, the operating pressure is now lower than the design pressure. For the remaining cases where the operating pressure still exceeds the design pressure, these conditions have been determined to be acceptable for the reasons stated above.

2. Piping Design and Operating Pressures for Relief Request RR-22, Item F
(Reference NRC RAI, Enclosure 2, Item 2, Section M)

NRC Question:

The requested information was not provided for relief request RR-22, item F; was this an editorial omission?

APCo Reply:

Yes. The requested information for RR-22, Item F has been added to the revised Attachment 7 (copy attached).

3. Holding Time for Hydrostatic Test of Steam Generator Secondary Side and Connecting Piping
(Reference NRC RAI, Enclosure 2, Item 2, Section 0)

NRC Question:

Relief request RR-30 requests a reduced holding time for hydrostatically testing the steam generator secondary side. Relief request RR-22, items A through D request relief from the hydrostatic test pressure. Since the portions of piping identified in RR-22 A-D are unisolable from the steam generator secondary side, is it intended that the reduced holding time addressed in RR-30 be applied to RR-22 A-D also?

APCo Reply:

Yes. Relief requests RR-22 and RR-30 will be revised to include notes which reference the complementing relief request and clarifying that the reduced holding time discussed in RR-30 applies to the unisolable piping discussed in RR-22 A-D.

ATTACHMENT 7

Relief Request No.	Line No.	Adj. Line No.	Op Press.	Design Press.	Code (1.25x) Hydro Press.	Relief Request Alt. Press.
RR-22 A	DCB-1A	S/G	1200	1600	2000	1344
	DCB-1B	S/G	1200	1600	2000	1344
	DCB-1C	S/G	1200	1600	2000	1344
RR-22 B	DBB-1	S/G	923	1600	2000	1344
RR-22 C	CBB-5	S/G	1000	1500	1875	1344
	CBB-6	S/G	1000	1500	1875	1344
	CBB-7	S/G	1000	1500	1875	1344
	CBB-8	S/G	1000	1500	1875	1344
	CBB-9	S/G	1000	1500	1875	1344
	CBB-10	S/G	1000	1500	1875	1344
RR-22 D	DBB-2	S/G	1270	1600	2000	1344
RR-22 F	CCB-18	HCB-20	2555	2485	3107	188
RR-23 A	CCB-27	RCS Loops	2235	2485	3107	2280
	CCB-33	RCS Loops	2235	2485	3107	2280
	CCB-34	RCS Loops	2235	2485	3107	2280
	CCB-38	RCS Loops	2235	2485	3107	2280
	CCB-39	RCS Loops	2235	2485	3107	2280
	CCB-40	RCS Loops	2235	2485	3107	2280
	CCB-41	RCS Loops	2235	2485	3107	2280
	CCB-42	RCS Loops	2235	2485	3107	2280
	CCB-43	RCS Loops	2235	2485	3107	2280
	CCB-44	RCS Loops	2235	2485	3107	2280
	CCB-47A	RCS Loops	2300	2485	3107	2280
	CCB-47B	RCS Loops	2300	2485	3107	2280
	CCB-47C	RCS Loops	2300	2485	3107	2280
RR-23 B	CCB-57	RCS Loops	2235	2485	3107	2280
RR-23 C	CCB-37	RCS Loops	2235	2485	3107	2280
	CCB-54	RCS Loops	2235	2485	3107	2280
	CCB-55	RCS Loops	2235	2485	3107	2280
	CCB-56	RCS Loops	2235	2485	3107	2280
RR-23 D	CCB-21	RCS Loops	2555	2485	3107	2280
	CCB-50	RCS Loops	2555	2485	3107	2280
RR-23 E	CCB-22	RCS Loops	2555	2485	3107	2280
RR-23 F	CCB-30	RCS Loops	2555	2485	3107	2280
RR-23 G	CCB-31	RCS Loops	2555	2485	3107	2280
RR-23 H	CCB-24	CCA-24	650	2485	3107	850
	CCB-53	CCA-24	650	2485	3107	850
	CCB-54	RCS Loops	650	2485	3107	850
RR-23 I	CCB-29	RCS Loops	600	2485	3107	2280
RR-23 J	CCB-22	RCS Loops	2555	2485	3107	2280
	CCB-32	RCS Loops	75	2485	3107	2280
RR-23 K	CCB-9	RCS Loops	2350	2485	3107	2280
RR-23 L	CCB-10	RCS Loops	2350	2485	3107	2280
RR-23 M	CCB-45	RCS Loops	2350	2485	3107	2280
RR-23 N	CCB-48A	RCS Loops	75	2485	3107	2280
	CCB-48B	RCS Loops	75	2485	3107	2280
	CCB-48C	RCS Loops	75	2485	3107	2280
RR-23 O	CCB-63	RCS Loops	2235	2485	3107	2280
RR-25	CCB-36	RV Flange Gasket	2235	2485	3107	2280 *
RR-29	CCB-62	HCC-263	2735	2485	3107	2750
RR-30	Steam Gen. Secon. Side	SEE RR-22 A,B,C,D	775	1085	1344	1344

* Only if RV flange gasket leaks.