NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	LICENSEE CODE: I L Q A D 1 (2)	LICENSE NUMBER: 0 0 0 - 0 0 0 - 0 0 0 (3)	LICENSE TYPE: 4 1 1 1 1 (4)	CAT (5)

CON'T 01	REPORT SOURCE: L (6)	DOCKET NUMBER: 0 5 0 0 0 2 5 4 (7)	EVENT DATE: 1 0 2 7 8 1 (8)	REPORT DATE: 1 1 1 0 8 1 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On October 27, 1981, while performing a surveillance test on the Electrohydraulic
03 Control low pressure scram pressure switches, pressure switch 1-560C-PS-3 tripped
04 at 876 psig. The Technical Specification Table 3.1-3 requirement is greater than
05 or equal to 900 psig. The other switches in the one-out-of-two-twice logic were
06 found to trip above 900 psig; therefore, the consequences of the occurrence are
07 minimal. The system would have initiated a Reactor scram within the Technical
08 Specification limits.

09	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
	I A (11)	E (12)	E (13)	I N S T R U (14)	E (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 1	—	0 2	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	A (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	N (25)	B 0 7 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this occurrence is instrument setpoint drift. The pressure switch
11 was immediately recalibrated and functionally tested. This instrument has had an
12 unsatisfactory history of drifting out of calibration and will be replaced.
13 A new instrument has been ordered from the vendor and will be replaced during the
14 next convenient outage following delivery.

15	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	E (28)	0 9 7 (29)	NA	B (31)	Routine Surveillance

16	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

17	NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

18	NUMBER	DESCRIPTION (41)
	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

19	TYPE	DESCRIPTION
	Z (42)	NA

PUBLICITY

20	ISSUED	DESCRIPTION (45)
	N (44)	NA

NRC USE ONLY

D. Hannum PHONE 309-654-2241, ext. 193

I. LER NUMBER: LER/RO 81-21/03L-0

II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station

III. FACILITY NAME: Unit One

IV. DOCKET NUMBER 050-254

V. EVENT DESCRIPTION:

On October 27, 1981, while performing surveillance test QIS-39, Loss of Electrohydraulic Control Fluid Pressure Scram, pressure switch 1-5700-PS-3 was found to trip at 876 psig. This is 14 psig below the 900 psig setpoint requirement of Technical Specification Table 3.1-3. The other three pressure switches were found to be within the Technical Specification limit. Work Request Q15465 was written to investigate the problem.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The four pressure switches are arranged in a one-out-of-two-twice logic to initiate a Reactor Protection System function upon loss of EHC pressure. Redundant switches were demonstrated to be within allowable limits; therefore, low EHC fluid pressure would have been sensed at the proper setpoint and a Reactor scram would have been initiated. Safe operation of the Reactor was not affected at any time.

VII. CAUSE:

The cause of this occurrence was instrument setpoint drift. The pressure switch is model C-9612-2, manufactured by Barksdale Company.

VIII. CORRECTIVE ACTION:

The immediate corrective action was to recalibrate the pressure switch setpoint. A functional test was then successfully performed to demonstrate the operability of the pressure switch. This pressure switch has a history of instrument drift and will be replaced like-for-like with a new pressure switch at a convenient outage following delivery of the new instrument from the vendor.