

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT NO. 81-10

OCTOBER, 1981

APPROVED BY:


STATION MANAGER

8111200464 811113
PDR ADDCK 05000280
R PDR

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-1-
OPERATING DATA REPORT

DOCKET NO. 50-280
DATE 11-09-81
COMPLETED BY Sue D. Dunn
TELEPHONE (804) 357-384

OPERATING STATUS

1. Unit Name: Surry Unit #1
2. Reporting Period: 10-01-81 through 10-31-81
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745.0	7296.0	77664.0
12. Number Of Hours Reactor Was Critical	617.1	2339.8	44878.3
13. Reactor Reserve Shutdown Hours	0	0	3731.5
14. Hours Generator On-Line	612.4	2277.5	43946.3
15. Unit Reserve Shutdown Hours	0	0	3736.2
16. Gross Thermal Energy Generated (MWH)	1474194.0	5278025.8	101667426.8
17. Gross Electrical Energy Generated (MWH)	466025.0	1686980.0	32988723.0
18. Net Electrical Energy Generated (MWH)	441630.0	1591181.0	31291105.0
19. Unit Service Factor	82.2%	31.2%	56.6%
20. Unit Availability Factor	82.2%	31.2%	61.4%
21. Unit Capacity Factor (Using MDC Net)	76.5%	28.1%	52.0%
22. Unit Capacity Factor (Using DER Net)	75.2%	27.7%	51.1%
23. Unit Forced Outage Rate	1.5%	5.1%	25.5%
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>Spring Maintenance - 02-19-82 - 10 days</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

-2-
OPERATING DATA REPORT

DOCKET NO. 50-281
DATE 11-09-81
COMPLETED BY Sue D. Dunn
TELEPHONE (804) 357-3184

OPERATING STATUS

1. Unit Name: Surry Unit #2
2. Reporting Period: 10-01-81 through 10-31-81
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745.0	7296.0	74544.0
12. Number Of Hours Reactor Was Critical	745.0	6864.8	44648.8
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	745.0	6824.4	43961.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1788172.0	16489466.4	102944127.4
17. Gross Electrical Energy Generated (MWH)	576755.0	5324420.0	33563414.0
18. Net Electrical Energy Generated (MWH)	545308.0	5042885.0	31821373.0
19. Unit Service Factor	100%	93.5%	59.0%
20. Unit Availability Factor	100%	93.5%	59.0%
21. Unit Capacity Factor (Using MDC Net)	94.4%	89.2%	55.1%
22. Unit Capacity Factor (Using DER Net)	92.9%	87.7%	54.2%
23. Unit Forced Outage Rate	0%	1.0%	17.3%
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	Refueling - 11-06-81 - 42 days		

25. If Shut Down At End Of Report Period. Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

Forecast

Achieved

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER, 1981DOCKET NO. 50-280UNIT NAME Surry Unit #1DATE 11-03-81COMPLETED BY Sue D. DunnTELEPHONE (804)357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-15	10-01-81	S	123.1	H	4				Continuation of shutdown for Auxiliary Feedwater System leak repairs which began 09-20-81.
81-16	10-06-81	F	9.5	H	3				"A" Main feed regulating valve stuck open while increasing power after startup causing trip on "A" S/G high level. The problem was corrected and the valve tested prior to reactor startup.
81-17	10-15-81	F	0.0	H	4				Loss of turbine power experienced due to problem with EHC System. Problem corrected and power returned to 100%.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER, 1981

DOCKET NO. 50-281
 UNIT NAME Surry Unit #2
 DATE 11-03-81
 COMPLETED BY Sue D. Dunn
 TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-15	10-11	F	0.0	H	4				Three broken wires on #1 GV Valve for the main turbine caused valve to go closed causing loss of turbine power from 655 MWe to 278 MWe. The wires were replaced.

¹
 F- Forced
 S- Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 C Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

(9/77)

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 1

MONTH: OCTOBER, 1981

<u>DATE</u>	<u>TIME</u>	<u>HOURS</u>	<u>LOAD, MW</u>	<u>REDUCTIONS, MW</u>	<u>MWH</u>	<u>REASON</u>
None during this reporting period.						
MONTHLY TOTAL					0	

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 2

MONTH: OCTOBER, 1981

<u>DATE</u>	<u>TIME</u>	<u>HOURS</u>	<u>LOAD, MW</u>	<u>REDUCTIONS, MW</u>	<u>MWH</u>	<u>REASON</u>
None during this reporting period.						
MONTHLY TOTAL					0	

DOCKET NO 50-280
UNIT SURRY I
DATE 11-1-81
COMPLETED BY SUE D. DUNN

AVERAGE DAILY UNIT POWER LEVEL

MONTH: OCTOBER 81

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.0	17	736.5
2	0.0	18	736.8
3	0.0	19	734.4
4	0.0	20	735.0
5	0.0	21	734.0
6	142.9	22	733.4
7	647.1	23	735.0
8	734.8	24	735.1
9	735.0	25	764.3
10	736.3	26	732.0
11	734.9	27	729.0
12	735.4	28	728.6
13	737.0	29	727.5
14	735.3	30	731.9
15	697.6	31	732.0
16	738.1		

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 % LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

DOCKET NO 50-281
UNIT SURRY II
DATE 11-1-81
COMPLETED BY SUE D. DUNN

AVERAGE DAILY UNIT POWER LEVEL

MONTH: OCTOBER 81

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	738.5	17	744.6
2	738.8	18	749.4
3	742.5	19	748.5
4	744.4	20	746.8
5	741.6	21	748.3
6	740.9	22	749.5
7	738.9	23	748.3
8	742.9	24	747.9
9	743.4	25	778.2
10	729.7	26	742.6
11	540.3	27	742.6
12	583.9	28	743.2
13	740.9	29	745.9
14	739.8	30	745.8
15	741.7	31	744.7
16	746.8		

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 % LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

SUMMARY OF OPERATING EXPERIENCE

OCTOBER, 1981

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT I

- October 1, - This reporting period begins with the unit at cold shutdown for repair of auxiliary feedwater system leaks.
- October 4, - Left cold shutdown condition ($>200^{\circ}\text{F}$) at 1905.
- October 5, - Left $350^{\circ}\text{F}/450$ PSIG at 0955. Reached hot shutdown condition (547°F) at 1912.
- October 6, - The reactor was critical at 0028. The generator was synchronized to the line at 0308. At 0313 a reactor trip occurred due to a high level in "A" Steam Generator. The high level was a result of control system problem with the "A" main feed regulating valve. The control system problem with the "A" MF regulating valve was repaired and the valve tested at 0610. The reactor was critical at 1035 and the generator was synchronized to the line at 1243. The rate of power increase was reduced to 3% per hour at 1400 with the unit at 30%.
- October 7, - The unit reached 100% power at 1200.
- October 15, - When closing the #3 Governor Valve for the main turbine for repair of the LVDT Core Rod for the position indication the valve closed immediately and the #4 Governor Valve did not open as designed. This caused a loss of approximately 200MWe. The LVDT Core Rod was replaced for #3 Governor Valve and the valve was re-opened at 0135. A power increase was commenced at 3% per hour and the unit reached 100% power at 1100.
- October 31, - This reporting period ends with the unit at 100% power.

UNIT II

- October 1, - This reporting period begins with the unit at 100% power.
- October 10, - Commenced decreasing power to 80% to remove "4B" feedwater heater from service for repairs. Stopped power decrease to 80% at 2225.
- October 11, At 1835 turbine power dropped from 655 MWe to 278 MWe when #1 Governor Valve went closed. Conditions were stabilized with the unit at 49% power and a power increase began at 1930. At 1958 #1 Governor Valve was found to be swinging. At 2003 all governor valves were swinging and turbine power was fluctuating from 580MWe to 245 MWe. Ran valve position limiter back to close #1 Governor Valve. Stabilized power at 51%, 350 MWe in turbine manual. At 2050 electro-hydraulic control

-10-
SUMMARY OF OPERATING EXPERIENCE

(CONTINUED)

UNIT II

fluid to #1 Governor Valve was isolated and instrument technicians were called in to find and correct problem with EHC System. At 2307 instrument technicians reported problem was three broken wires on #1 Governor Valve. The wires were replaced and valve position indication for #1 Governor Valve verified correct at 2345.

- October 12 - Opened #1 Governor Valve at 0005 and #3 Governor Valve closed back to compensate. At 0027 commenced raising power to 80% at 150 MWe per hour. Stopped power increase at 80% power, 615 MWe at 0219. Commenced a power increase at 2121 after returning "4B" feedwater heater to service. The unit reached 100% power at 2307.
- October 30, - Commenced VOPEX Emergency Plan Drill at 1044. Secured from VOPEX drill for the day at 1730.
- October 31, - VOPEX Drill recommenced at 0500. VOPEX Drill terminated at 1525. This reporting period ends with the unit at 100% power.

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

OCTOBER, 1981

None during this reporting period.

FACILITY CHANGES REQUIRING
NRC APPROVAL

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

OCTOBER, 1981

D/C 80-86 - Modification of Masonary Walls

Unit

The following portions of this design change were implemented 1 & 2
in response to IE Bulletin 80-11.

D/C 80-86A	Modification of Masonary Walls	AB-45-10-6 and AB-45-10-9.
D/C 80-86B	Modification of Masonary Wall	SP-27-0-10.
D/C 80-86C	Modification of Masonary Wall	AB-27-6-1.
D/C 80-86E	Modification of Masonary Walls	AB-27-6-33 and AB-27-6-34.
D/C 80-86G	Modification of Masonary Wall	AB-27-6-32.
D/C 80-86I	Modification of Masonary Walls	IS-21-2-1 and IS-21-2-2.
D/C 80-86J	Modification of Masonary Wall	AB-27-6-35.
D/C 80-86K	Modification of Masonary Walls	AB-15-0-1 and AB-15-0-4.
D/C 80-86L	Modification of Masonary Walls	SB-9-6-1, 2, 3, 14, 15, 16, and 17.
D/C 80-86M	Modification of Masonary Walls	SB-9-6-6 and SB-9-6-8.
D/C 80-86N	Modification of Masonary Walls	SB-9-6-11, 12 and AB-27-6-8.
D/C 80-86P	Modification of Masonary Wall	SB-27-0-3.
D/C 80-86Q	Modification of Masonary Walls	SB-27-0-8 and AB-45-10-21.
D/C 80-86R	Modification of Masonary Wall	AB-27-6-7.
D/C 80-86S	Modification of Masonary Walls	AB-27-6-19 and AB-27-6-20.

Summary of Safety Analysis

The completed modifications will have no effect on the operation
of safety related equipment.

D/C 81-01 - Addition of Isolation Valve to Aire Start System

Unit

This design change added a 1 1/2 in. isolation valve to each of 1 & 2
the air compressors of the air start system for the emergency
diesels. The isolation valves in each of the compressor
discharge lines are downstream of the check valves. This will
allow work on the check valves without removing any of the air
supply from service.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

OCTOBER, 1981

D/C 81-01 - Summary of Safety Analysis

Unit

This modification installed the isolation valves in close proximity of a support, therefore, negating any increased seismic loading of the pipe. As a result of this design change, there is no effect on the operation of any safety related equipment.

1 & 2

D/C 81-17 - Reserve Station Service Load Shed Scheme

Unit

This design change installed a load shedding scheme to reduce the margin of overload on the RSS system in the event of simultaneous loading of both units on the RSS system. The automatic scheme includes manual override. This design change is an interim change pending resolution of a permanent solution to present NRC GDC17 guidelines.

1 & 2

Summary of Safety Analysis

This modification improves the voltage available from the offsite power source under certain postulated conditions. There are no changes to system design or operation for any safety or non-safety equipment.

TESTS AND EXPERIMENTS REQUIRING
NRC APPROVAL

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

TEST AND EXPERIMENTS THAT
DID NOT REQUIRE NRC APPROVAL

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL
September, 1981
(continued)

D/C 79-41 New Fuel Elevator Modification 1

This design change installed a key lock switch on the up button for the new fuel elevator to prevent raising the elevator without the key which is under the control of the Shift Supervisor.

SUMMARY OF SAFETY ANALYSIS

This modification provides an increased margin of safety during fuel handling operations.

D/C 79-69 Auxiliary Shutdown Panel/Main Control Board Isolation Modification 1

This design change provides complete electrical isolation between circuits in the Main Control Board and the Auxiliary Shutdown Panel as required by the NRC issued, Fire Protection Safety Evaluation Report, dated September 19, 1979.

SUMMARY OF SAFETY ANALYSIS

This modification does not create a safety problem, since this upgrades safety related circuits needed for safe shutdown. This modification does not degrade the functioning of any other safety related systems.

* D/C 80-53 Insert Check Valve Replacement Bearings 1

This design change provided a method for replacing teflon insert check valve bearings with stainless steel bearings. The teflon bearings were not qualified for the estimated total integrated radiation dose due to the valve locations. The overexposure of these valve bearings could have resulted in failure of the valve to operate properly.

SUMMARY OF SAFETY ANALYSIS

The Service Water System and Component Cooling Water System capability and operation readiness is not affected. There are no safety implications created by this design change. This modification increases the reliability of the check valve to operate after an accident.

D/C 80-55 Replace Mechanical Seal-Charging Pump Cooling Water Pump 1 & 2

This design change replaced the mechanical seals on the Charging Pump Cooling Water Pumps secondary seals. The teflon secondary seals were not qualified to the 40 year normal plus post-accident integrated radiation doses. The replacement material (ethylene propylene rubber) meets or exceeds the environmental requirements for the original teflon seals.

OTHER CHANGES, TESTS AND EXPERIMENTS

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

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CHEMISTRY REPORT

OCTOBER __, 19 81

T.S. 6.6.3.J

PRIMARY COOLANT ANALYSIS	UNIT NO. 1 (D)			UNIT NO. 2 (C)		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., $\mu\text{Ci/ml}$	2.31° (B)	9.44^{-2}	1.05°	2.80^{-1}	1.20^{-1}	2.06^{-1}
Suspended Solids, ppm	0.1	0.1	0.1	0.1	0.1	0.1
Gross Tritium, $\mu\text{Ci/ml}$	2.16^{-1}	6.96^{-2}	1.43^{-1}	1.18^{-1}	1.04^{-1}	1.10^{-1}
Iodine-131, $\mu\text{Ci/ml}$	$2.63^{(B)-1}$	3.60^{-2}	1.43^{-1}	9.59^{-3}	2.72^{-3}	4.49^{-3}
I-131/I-133	.7479	.2069	.4505	$2.0200^{(E)}$.8416	1.4369
Hydrogen, cc/kg	46.6	4.9 (A)	27.7 (A)	57.0	32.4	45.4
Lithium, ppm	1.67	0.47	1.22	.96	.34	.64
Boron-10, ppm +	389.84	162.68	183.84	28.03	7.64	15.67
Oxygen-16, ppm	4.40 (G)	.000	1.67	.000	.000	.000
Chloride, ppm	<.05	<.05	<.05	<.05	<.05	<.05
pH @ 25°C	7.76	5.50	6.48	7.86	5.95	7.38

+ Boron-10 = Total Boron x 0.196

NON-RADIOACTIVE CHEMICAL
RELEASES, POUNDS (F)
T.S. 4.13.A.6

Phosphate	-	Boron	504
Sulfate	-	Chromate	.10
50% NaOH	-	Chlorine	-0-

Remarks: (A) Reflects Unit #1 startup 10-6-81 (B) High values due to suspected fuel failures

(C) Unit rampdown to 50% power 10-10-81 for feedwater heater repairs.

(D) Unit start-up 10-6-81 - Unit runback from 100% to 71% power.

(E) High values due to suspected pin-hole leaks in fuel.

(F) These levels of chemicals should represent no major adverse environmental impact.

(G) Unit at cold Shutdown.

DESCRIPTION OF ALL INSTANCES WHERE
THERMAL DISCHARGE LIMITS WERE EXCEEDED

OCTOBER, 1981

Due to the impairment of the circulating water system on the following days, the thermal discharge limits were exceeded as noted.

October 7, 1981	Exceeded 15.0°F ΔT across station*
October 8, 1981	Exceeded 15.0°F ΔT across station*
October 9, 1981	Exceeded 15.0°F ΔT across station
October 10, 1981	Exceeded 15.0°F ΔT across station*
October 11, 1981	Exceeded 15.0°F ΔT across station*
October 12, 1981	Exceeded 15.0°F ΔT across station*
October 13, 1981	Exceeded 15.0°F ΔT across station
October 14, 1981	Exceeded 15.0°F ΔT across station
October 15, 1981	Exceeded 15.0°F ΔT across station*
October 16, 1981	Exceeded 15.0°F ΔT across station*
October 17, 1981	Exceeded 15.0°F ΔT across station*
October 18, 1981	Exceeded 15.0°F ΔT across station
October 19, 1981	Exceeded 15.0°F ΔT across station
October 20, 1981	Exceeded 15.0°F ΔT across station
October 21, 1981	Exceeded 15.0°F ΔT across station
October 24, 1981	Exceeded 15.0°F ΔT across station*
October 25, 1981	Exceeded 17.5°F ΔT across station
October 26, 1981	Exceeded 15.0°F ΔT across station*
October 27, 1981	Exceeded 17.5°F ΔT across station*
October 28, 1981	Exceeded 15.0°F ΔT across station
October 29, 1981	Exceeded 15.0°F ΔT across station
October 31, 1981	Exceeded 15.0°F ΔT across station

*Indicates dates where station ΔT was 15.0°F or less across station for sometime during the day.

The ΔT excursions were allowable under Technical Specification 4.14.B.2. There were no reported instances of adverse environmental impact.

FUEL HANDLING

OCTOBER, 1981

Sixty-eight (68) new fuel assemblies and Sixty (60) new BPRA's were received this month for Unit 2 Cycle 6 refueling.

[illegible]

FUEL HANDLING

OCTOBER, 1981

DATE SHIPPED/RECEIVED	NO. OF ASSEMBLIES PER SHIPMENT	ANSI NO. INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
10-15-81	12	LM05XY/3.6	<2.5 MR/hr.
		LM05Y0/3.6	<2.5 MR/hr.
		LM05XZ/3.6	<2.5 MR/hr.
		LM05Y3/3.6	<2.5 MR/hr.
		LM09PG/3.6	<2.5 MR/hr.
		LM05YB/3.6	<2.5 MR/hr.
		LM05X4/3.6	<2.5 MR/hr.
		LM09PE/3.6	<2.5 MR/hr.
		LM05YA/3.6	<2.5 MR/hr.
		LM05XA/3.6	<2.5 MR/hr.
		LM05XF/3.6	<2.5 MR/hr.
		LM05XH/3.6	<2.5 MR/hr.
10-22-81	12	LM05YG/3.6	<2.5 MR/hr.
		LM05XX/3.6	<2.5 MR/hr.
		LM05JX/3.6	<2.5 MR/hr.
		LM05XE/3.6	<2.5 MR/hr.
		LM05XB/3.6	<2.5 MR/hr.
		LM05XU/3.6	<2.5 MR/hr.
		LM05YH/3.6	<2.5 MR/hr.
		LM05XQ/3.6	<2.5 MR/hr.
		LM05YK/3.6	<2.5 MR/hr.
		LM05XD/3.6	<2.5 MR/hr.
		LM05XV/3.6	<2.5 MR/hr.
		LM05Y7/3.6	<2.5 MR/hr.

FUEL HANDLING

OCTOBER, 1981

DATE SHIPPED/RECEIVED	NO. OF ASSEMBLIES PER SHIPMENT	ANSI NO. INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
10-28-81	12	LM09PC/3.6	<2.5 MR/hr.
		LM05X3/3.6	<2.5 MR/hr.
		LM05YF/3.6	<2.5 MR/hr.
		LM05Y2/3.6	<2.5 MR/hr.
		LM05XG/3.6	<2.5 MR/hr.
		LM05XT/3.6	<2.5 MR/hr.
		LM05YN/3.6	<2.5 MR/hr.
		LM05YJ/3.6	<2.5 MR/hr.
		LM05YP/3.6	<2.5 MR/hr.
		LM05YQ/3.6	<2.5 MR/hr.
		LM05X7/3.6	<2.5 MR/hr.
		LM05YC/3.6	<2.5 MR/hr.
10-30-81	12	LM09P9/3.6	<2.5 MR/hr.
		LM05XK/3.6	<2.5 MR/hr.
		LM09PH/3.6	<2.5 MR/hr.
		LM05Y4/3.6	<2.5 MR/hr.
		LM05X9/3.6	<2.5 MR/hr.
		LM05XR/3.6	<2.5 MR/hr.
		LM05X8/3.6	<2.5 MR/hr.
		LM09PA/3.6	<2.5 MR/hr.
		LM05X6/3.6	<2.5 MR/hr.
		LM05XC/3.6	<2.5 MR/hr.
		LM05Y1/3.6	<2.5 MR/hr.
		LM05Y8/3.6	<2.5 MR/hr.

PROCEDURE REVISIONS THAT CHANGED THE
OPERATING MODE DESCRIBED IN THE FSAR

OCTOBER, 1981

None during this reporting period.

DESCRIPTION OF PERIODIC TESTS WHICH WERE NOT
COMPLETED WITHIN THE TIME LIMITS
SPECIFIED IN TECHNICAL SPECIFICATIONS

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

INSERVICE INSPECTION

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

REPORTABLE OCCURRENCES PERTAINING TO
ANY OUTAGE OR POWER REDUCTIONS

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Mechanical Maintenance

DEPT-MECH

UNIT 1
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

REF/SERVIC	SY	COMP	NARKNO	SUMMARY	WKT/HR	U	NR	TOTL WRTN
10/01/81	SR	PURPS	R9-P-19	CHECK MECH SEALS ON PUMPS	SMALL CORD ACCESSIBLE AT THIS TIME	1	102091816	4464
10/01/81	FW	MOV	MOV-FW-151E	VALVE HAS BODY BONNET LEAK	REPLACED GASKET	1	109300130	45
10/01/81	CH	PUMP	1-CH-P-1H	INSTALL VENT KIG	REMOVED VENT AFTER INSTALL FOR TEST	1	110011230	5
10/02/81	CH	VALVE	1-CH-77	2 VALVE STEM MISSING	RENEWED BONNET ASSY	1	108301742	100
10/02/81	CS	MOV	MOV-202B	VALVE LEAKS PAST SEAT	OPENED AND CHECKED VALVE INTERNALS	1	109061022	720
10/03/81	SW	FLOW IND	F1-SW-100A	FLOW INDICATOR LEAKS EXCESS	LEVELD FLOW INDICATOR	1	109301600	67
10/03/81	SW	VALVE	1-SW-167	UNABLE TO CYCLE VALVE	COMPLETE	1	109302131	52
10/04/81	CS	PUMP	1-CS-P-1A	INBOARD MECH SEAL LEAKS	REPLACED MECH SEAL	1	107080614	48
10/04/81	CS	PUMP	1-CS-P-1A	MECH SEALS BOTH LEAKING	REPAIR MECH SEAL	1	110031330	27
10/04/81	SI	VALVE	1-SI-241	UNBOLT AND REMOVE BONNET	OPENED THEN REASSEMBLE	1	110031649	25
10/04/81	CS	PUMP	1-CS-P-1H	COUPLE PUMP TO MOTOR	BOLTED COUPLING	1	110041115	7
10/07/81	SW	PIPING	2-SW-78-136	FIBER GLASS LINE LEAKING	REPLACED ELBOW	1	109251059	293
10/07/81	SW	PUMP	1-SW-P-1C	REPACK	REPACKED	1	110051405	37
10/07/81	SW	PUMP	1-SW-P-10B	REPACK	ADDED PACKING	1	110051406	31

DEPT TOTAL

5921

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Mechanical Maintenance

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING
OUTAGE OR REDUCED POWER PERIODS

UNIT #2

MECHANICAL MAINTENANCE

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Electrical Maintenance

DEPT=ELEC

UNIT1
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RESEVRY	SYS	COMP	MARKNO	SUMMARY	WKPURF	U	HR	TOTWNTM
10/01/81	KC	VALVE	HCV-1556A	DISC * RUCOH ELEC T LEADS	COMP VALVE SAT	1	109281800	45
10/01/81	FW	MOV	MOV-FW-151F	MOV INDICATE CLOSED LOCALLY	COMPLETE VALVE NOT WORKS SAT	1	110011750	5
10/04/81	CS	PUMP MTR	1-CS-P-1A	CHECK MOTOR	BRIDGE * MEGGER MOTOR TOOK AMP READI	1	110041012	1
DEPT TOTAL								51

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Electrical Maintenance

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING
OUTAGE OR REDUCED POWER PERIODS

UNIT #2

ELECTRICAL MAINTENANCE

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Instrument Maintenance

DEPT-INST

UNITY
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

REFERENCE	SYS	COMP	MARKNO	SUMMARY	WHERE	U	NR	TOT/WTM
10/01/81	MS	INSTR	FI-1-494	CHANNEL FI-1-494 DOES NOT TRACK	REPLACED XMTR	1	109201205	144
10/06/81	RC	MONITOR	A	TIC 3 DISABLED	CHANGED INPUT	1	107030031	2278
10/06/81	RC	MONITOR		PRESSURE POINT READING LOW	P2 INPUT NOT USED	1	107111401	1944
DEPT TOTAL								4504

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Instrument Maintenance

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING
OUTAGE OR REDUCED POWER PERIODS

UNIT #2

INSTRUMENT MAINTENANCE

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.

HEALTH PHYSICS

OCTOBER, 1981

There was no single release of radioactivity or radiation exposure specifically associated with an outage that accounted for more than 10% of the allowable annual values in 10CFR20.

PROCEDURE DEVIATIONS REVIEWED BY STATION NUCLEAR
SAFETY AND OPERATING COMMITTEE AFTER TIME LIMITS
SPECIFIED IN TECHNICAL SPECIFICATIONS

OCTOBER, 1981

NONE DURING THIS REPORTING PERIOD.