NRC FORM 366
(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | F L C R P 3 | (2) | 0 0 - 0 0 0 0 0 - 0 0 | (3) | 4 1 1 1 1 | (4) | | | (5)
LICENSEE CODE — LICENSE NUMBER — LICENSE TYPE — CAT

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0 1 REPORT SOURCE | L | (6) | 0 5 0 - 0 3 0 2 | (7) | 1 0 3 0 8 1 | (8) | | (9)
DOCKET NUMBER — EVENT DATE — REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 At 1515 during refueling operations it was discovered that the FSAR analysis of the
0 3 Steam Generator Tube Rupture Accident was not conservative. This created an event
0 4 reportable under T.S. 6.9.1.8.h. There was no effect upon the health or safety of
0 5 the general public. This was the first event of this type.
0 6
0 7
0 8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	Z Z (11)	X (12)	Z (13)	Z Z Z Z Z Z (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 1	—	0 7 1	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	X (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	Z (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event is the FSAR assumption for this accident that Main Steam
1 1 Safety Valves will not lift following a reactor trip. If they do lift, the radio-
1 2 activity released will exceed FSAR calculations for this accident. CR-3 is in com-
1 3 pliance with 10 CFR 100 requirements when compared to worst case release. Appropriate
1 4 analysis has been initiated to determine the full effect of this postulated release.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	H (28)	0 0 0 (29)	NA	C (31)	Engineer observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

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PDR ADOCK 05000302
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PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

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(SEE ATTACHED SUPPLEMENTARY INFORMATION SHEET)

GPO 917-926

SUPPLEMENTARY INFORMATION

Report No.: 50-302/81-071/01T-0

Facility: Crystal River Unit 3

Report Date:

Occurrence Date: October 30, 1981

Identification of Occurrence:

An error discovered in an accident analysis described in the Final Safety Analysis Report could have permitted reactor operation in a manner less conservative than assumed in the analysis for this accident. This is reportable per Technical Specification 6.9.1.8.h.

Conditions Prior to Occurrence:

Mode 6 refueling (0%).

Description of Occurrence:

At 1515 during refueling operations, it was discovered that the Final Safety Analysis Report (FSAR) analysis of the Steam Generator Tube Rupture Accident was not conservative.

Designation of Apparent Cause:

The cause of this event is attributed to the Final Safety Analysis Report assumption that Main Steam Safety Valves will not lift following a reactor trip. If they do lift, it will cause a release of activity to the environment greater than that calculated by the FSAR for this accident.

Analysis of Occurrence:

There was no effect upon the health or safety of the general public. It is highly probable that Crystal River Unit 3 is in compliance with the offsite dose requirements for this accident specified in 10 CFR 100.

Corrective Action:

Appropriate analysis has been initiated.

Failure Data:

This is the first event of this type.

/rc