U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 | ALBRF 3 (2) | 00-00000-00 (3) | 4 1 1 1 1 (4) | (5)

8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

VT

1 | REPORT SOURCE L (6) | 05000296 (7) | 10 11 82 (8) | 11 09 82 (9)

8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

2 | Following a rod pattern adjustment, the calculated "R" factor (RRP/CMFLPD) (0.982)

3 | was less than the "R" factor used to set the APRM scram and rod block (1.0) for

4 | 2 hours and 15 minutes (T.S. 2.1.A.1.b & d). There was no effect on public health

5 | or safety. The calculated scram value was not exceeded during this time.

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8 9 80

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
IA (11)	X (12)	Z (13)	ZZZZZZ (14)	Z (15)	Z (16)
9 10	11	12	13 18	19	20

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	82	—	048	/	03	L	—	0
	21 22	23	24 26	27	28 29	30	31	32

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	G (19)	Z (20)	Z (21)	0000	Y (23)	N (24)	Z (25)	Z 999 (26)
33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 | Event was due to the insertion of control rods required to maintain operation

2 | with prescribed load line. Action was taken to revise the control rod pattern.

3 | "R" was within limits in 2 hours and 15 minutes. Tech specs have been submitted

4 | for units 2 and 3 (similar to present unit 1 tech specs) to allow 6 hours

5 | to correct "R".

6 7 80

FACILITY STATUS E (28) | % POWER 085 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY B (31) | DISCOVERY DESCRIPTION (32) Engineer observation

ACTIVITY CONTENT

RELEASED OF RELEASE Z (33) | Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

PERSONNEL EXPOSURES

NUMBER 000 (37) | TYPE Z (38) | DESCRIPTION (39) NA

PERSONNEL INJURIES

NUMBER 000 (40) | DESCRIPTION (41) NA

LOSS OF OR DAMAGE TO FACILITY

TYPE Z (42) | DESCRIPTION (43) NA

PUBLICITY

ISSUED N (44) | DESCRIPTION (45)

8211220279 821109
PDR ADOCK 05000296
S PDR

NRC USE ONLY

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Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 82048 Technical Specification Involved 2.1.A.1.b&d

Reported Under Technical Specification 6.7.2.b.(2)* Date Due NRC 11/10/82

Event Narrative:

Unit 1 was operating at 97-percent power; unit 2 was in a refueling outage. These units were unaffected by this event. Unit 3 was operating at 85-percent power following a control rod pattern adjustment. The unit was prevented from returning to full power following the control rod pattern due to feed pump problems. The delay at low flow caused a gradual load line increase as xenon burned out. When control rods were inserted as required to reduce the load line, "R" was reduced to 0.982 at 0122. This condition was in violation of Technical Specification 2.1.A.1.b and 2.1.A.1.d since the "R" factor setpoint was 1.0. The rod pattern was readjusted and the "R" was within limits (1.008) at 0337. There was no effect on public health or safety. The calculated APRM scram value was not exceeded during this event. A technical specification revision has been proposed and submitted to the NRC to allow up to six hours for "R" factor being less than the "R" setpoint as currently permitted by unit 1 technical specifications.

* Previous Similar Events:

BFRO-50-259/82019
260/81037, 82022
296/81045, 81041, 81040, 82038

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

Revised: J.R.P.