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OPERATING DATA REPORT

DOCKET NO. 50-266

DATE November 8, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH UCLEAR PLANT UNIT 1 NOTES .
2. REPORTING PERIOD: OCTOBER 1982
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): 390.0
10. REASONS FOR RESTRICTIONS, (IF ANY): Power level restricted because of self-imposed
hot leg temperature limitation in an attempt to limit steam generator tube corrosion.
THIS MONTH YR TO DATE CUMULATIVE
11. HOURS IN REPORTING PERIOD 745 7,296 105,072
12. NUMBER OF HOURS REACTOR WAS CRITICAL 515.3 6,642.4 86,995.6
13. REACTOR RESERVE SHUTDOWN HOURS 0.0 11.4 618.7
14. HOURS GENERATOR ON LINE 505.6 6,602.1 84,574.3
15. UNIT RESERVE SHUTDOWN HOURS 0.0 27.4 791.7
16. GROSS THERMAL ENERGY GENERATED (MWH) 595,125 7,996,075 115,348,915
17. GROSS ELECTRICAL ENERGY GENERATED (MWH) 196,200 2,656,150 33,677,430
18. NET ELECTRICAL ENERGY GENERATED (MWH) 184,637 2,515,783 36,796,994
19. UNIT SERVICE FACTOR 67.9 90.5 80.5
20. UNIT AVAILABILITY FACTOR 67.9 90.9 81.2
21. UNIT CAPACITY FACTOR (USING MDC NET) 50.1 69.7 71.7
22. UNIT CAPACITY FACTOR (USING DER NET) 49.9 69.4 70.5
23. UNIT FORCED OUTAGE RATE 0.0 0.2 2.8
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
NONE

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: DECEMBER 10, 1982

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE November 8, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
2. REPORTING PERIOD: OCTOBER 1982
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745	7,296	89,857
12. NUMBER OF HOURS REACTOR WAS CRITICAL	741.4	6,203.7	80,594.1
13. REACTOR RESERVE SHUTDOWN HOURS	1.3	3.8	196.8
14. HOURS GENERATOR ON LINE	737.4	6,132.6	79,191.2
15. UNIT RESERVE SHUTDOWN HOURS	1.6	3.2	181.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,094,393	9,060,327	109,317,171
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	368,190	3,039,930	37,062,940
18. NET ELECTRICAL ENERGY GENERATED (MWH)	351,325	2,898,854	35,282,320
19. UNIT SERVICE FACTOR	99.0	84.1	88.1
20. UNIT AVAILABILITY FACTOR	99.2	84.1	88.3
21. UNIT CAPACITY FACTOR (USING MDC NET)	95.3	80.3	79.9
22. UNIT CAPACITY FACTOR (USING DER NET)	94.9	79.9	79.0
23. UNIT FORCED OUTAGE RATE	0.0	0.1	1.5
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Eleven-week refueling outage scheduled to begin March 25, 1983.			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

DOCKET NO. 50-266

UNIT NAME Point Beach Unit 1

DATE November 8, 1982

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH October, 1982

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>364</u>	11	<u>363</u>	21	<u>373</u>
2	<u>369</u>	12	<u>371</u>	22	<u>- 10</u>
3	<u>346</u>	13	<u>371</u>	23	<u>- 8</u>
4	<u>368</u>	14	<u>370</u>	24	<u>- 2</u>
5	<u>368</u>	15	<u>369</u>	25	<u>- 2</u>
6	<u>369</u>	16	<u>370</u>	26	<u>- 2</u>
7	<u>370</u>	17	<u>370</u>	27	<u>- 2</u>
8	<u>370</u>	18	<u>368</u>	28	<u>- 2</u>
9	<u>371</u>	19	<u>371</u>	29	<u>- 2</u>
10	<u>358</u>	20	<u>371</u>	30	<u>- 2</u>
				31	<u>- 2</u>

DOCKET NO. 50-301

UNIT NAME Point Beach Unit 2

DATE November 8, 1982

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH October, 1982

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>277</u>	11	<u>486</u>	21	<u>486</u>
2	<u>461</u>	12	<u>473</u>	22	<u>487</u>
3	<u>427</u>	13	<u>486</u>	23	<u>487</u>
4	<u>442</u>	14	<u>487</u>	24	<u>487</u>
5	<u>485</u>	15	<u>483</u>	25	<u>487</u>
6	<u>486</u>	16	<u>470</u>	26	<u>487</u>
7	<u>488</u>	17	<u>475</u>	27	<u>488</u>
8	<u>483</u>	18	<u>466</u>	28	<u>486</u>
9	<u>486</u>	19	<u>477</u>	29	<u>488</u>
10	<u>473</u>	20	<u>452</u>	30	<u>487</u>
				31	<u>487</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1982DOCKET NO. 50-266UNIT NAME Point Beach Unit¹DATE November 8, 1982COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code	Cause and Corrective Action To Prevent Recurrence
5	821022	S	238.4	C	1	N/A	ZZ	ZZZZZZ	Began seven-week refueling outage.

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1982DOCKET NO. 50-301UNIT NAME Point Beach Unit' 2DATE November 8, 1982COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting ³ Down Reactor	Licensee Event Report No.	System Code ⁴	Component Code	Cause and Corrective Action To Prevent Recurrence
5	820925	S	7.6	B	1	N/A	CB	PUMPXX	No. 2 seal on "B" reactor coolant pump exhibited signs of increased degradation and was replaced.

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266
Unit Name Point Beach Unit 1
Date November 8, 1982
Completed By C. W. Fay
Telephone 414/277-2811

Unit 1 operated at 368 MWe net from October 1, 1982 until October 22, 1982, when after 194 days of continuous operation it was shut down for its tenth refueling outage. During the reporting period while the unit was operating, the primary-to-secondary leakage was less than 10 gallons per day.

At 0447 hours on October 2, 1982, valve 1CV-3200C failed to close during a surveillance test which is performed quarterly. This valve is one of two valves in series in the suction of the containment radioactive gas and particulate sampling system. Valve CV-3200C is located inside containment and receives a containment isolation signal along with series valve CV-3200B, which is located outside containment.

The problem was identified as a stuck solenoid, which prevented the air pressure from bleeding off the valve operator and subsequently prevented the valve from closing. The problem was corrected and the valve was cycled several times without further problems and the system returned to normal. The valve failure constitutes a violation of Technical Specification 15.3.6 and a 14-day Licensee Event Report is currently being drafted to be filed with the NRC. The Resident Inspector has been notified of this event.

On October 15, 1982, at 0145 hours, while calibrating 1R15 (Unit 1 air ejector radiation monitor) in the Unit 1 turbine hall, a leak occurred in the connection fittings. This resulted in the release of about 1.2 cubic feet of radioactive gas to the Unit 1 turbine hall. A total of about 0.16 Curies were released to the turbine hall. A red phone notification of the event was made. The release rate was calculated to be 0.5% of the annual average allowable. This event was not considered reportable.

Also on October 15, 1982, a turbine runback reduced load to 320 MWe net for one hour. The runback resulted when due to a misunderstanding, the wrong instrument bus was shifted. In order to investigate an arcing problem with motor-generator set 1GY04, the yellow bus was supposed to be shifted to its alternate power supply. Instead the white bus was shifted, and before it was noted and corrected, 1GY04 was tripped causing a runback. The white bus was returned to its normal power supply, the yellow bus shifted to its alternate and the unit returned to full power. An additional result of the lost bus was the loss of 1PT-950, Unit 1 containment pressure. The loss of this channel for approximately one minute reduced the degree of redundancy required by Technical Specification 15.3.5-3 to zero. Both sets of containment spray logic were operable at all times; one with 3 out of 3, one with 2 out of 3 channels available.

This event of lost redundancy is reportable in accordance with Technical Specification 15.6-9.2.A.2. The Resident Inspector has been notified of the event and Licensee Event Report 82-018/01T-0 will be sent to the NRC describing the event.

During the shutdown, the secondary sides of Unit 1 "A" & "B" steam generators were leak tested. Two explosive plugged tubes in the "A" steam generator had leakage rates of $1\frac{1}{2}$ and 2 drops per minute while one explosive plugged tube in the "B" steam generator leaked approximately one drop per minute and two others were visibly wet.

Eddy current inspection results of the primary side of the steam generators revealed 7 tubes with pluggable indications: Four tubes in "A" steam generator and 3 tubes in "B" steam generator. Mechanical plugging and a closeout inspection was completed on the steam generators and a 24-hour written notification of the degraded tubes was made to the NRC under Licensee Event Report No. 82-017/01T-0.

During performance of inservice inspection related activities on Unit 1 valve 559B, "B" loop RTD bypass manifold isolation was found to have a body-to-bonnet leak. Four of 12 body-to-bonnet studs were corroded; one losing approximately 50% diameter. It is believed that this leak was the small, difficult to find leak that has plagued Unit 1 containment for some time. This event constitutes a reportable event as an abnormal degradation of the reactor coolant system boundary.

Other work completed in conjunction with the outage was the overhaul of the "A" reactor coolant pump motor, safety valve inspections, moisture separator reheater inspections, turbine generator overhaul, "B" reactor coolant pump motor and seal inspection and the changeout of the air-operated isolation valves of the steam generator blowdown line.

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-301
Unit Name Point Beach Unit 2
Date November 8, 1982
Completed By C. W. Fay
Telephone 414/277-2811

Heatup of the primary coolant was accomplished on Unit 2 following reactor coolant pump seal replacement and at 0335 hours on October 1, 1982, the reactor was taken critical. The turbine was phased on line at 0737 hours and achieved full power 5 hours later. There were no other significant load reductions during the report period.

During restoration of the Unit 2 low steam header safety injection actuation bistables, a significant voltage perturbation occurred while attempting to replace a blown fuse on 2PC-483AX in the red instrument rack. The voltage perturbation was the result of a relay failure. The relay was replaced and tested satisfactorily on October 1, 1982. This event was not considered reportable.

On October 5, 1982, while returning to service the Unit 2 letdown gas stripper, an undetermined number of diaphragm valves began leaking primary coolant into the auxiliary building. The inlet valve to the letdown gas stripper (2GS-3) was closed when the stripper was placed in service which caused the line to pressurize to the relief valve setpoint. The NRC was notified of the event via the red phone. The maximum discharge rate amounted to .632% of the Technical Specification 15-minute release rate. A significant operating event report is currently being prepared to cover this event.

On October 6, 1982, Unit 2 main steam line pressure transmitter, 2PT-478, setpoint was discovered low during routine surveillance testing. This transmitter actuates a safety injection signal upon a low steam line pressure. The transmitter was recalibrated and is operating satisfactorily. This event is reportable in accordance with Technical Specification 15.3.5-1 and a 30-day Licensee Event Report will be filed with the NRC.

SUPPLEMENT TO MONTHLY REPORT FOR October 1982

REFUELING INFORMATION REQUEST

In accordance with our letter dated February 21, 1978, which provided certain refueling information, we are providing the following update to that refueling information: (NC = No Change)

a) Next Scheduled Refueling Shutdown:	Unit 1: <u>09/30/83</u>
	Unit 2: <u>03/25/83</u>
b) Scheduled Date for Restart:	Unit 1: <u>03/30/84</u>
	Unit 2: <u>06/10/83</u>
c) License Amendment Required/Staff 10 CFR 50.59 Review Completed:	Unit 1: <u>Yes, steam generator replacement</u>
	Unit 2: <u>Yes, steam generator sleeving</u>
d) Scheduled Date for Submitting Supporting Information:	Unit 1: <u>Submitted</u>
	Unit 2: <u>Submitted</u>
e) Important Licensing Considerations:	Unit 1: <u>Steam generator replacement</u>
	Unit 2: <u>Steam generator sleeving</u>
f) Number of Fuel Assemblies in Storage Pool:	<u>396*</u>
g) Other:	

*Includes 20 spent fuel assemblies from the Unit 1 fall 1982 refueling.