AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE November 3, 1982
COMPLETED BY R. W. Short
TELEPHONE (402) 536-4543

MONTH October 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	404.5	17	308.8
2	406.5	18	309.5
3	406.5	19	309.6
4	406.6	20	306.0
5	405.8	21	308.0
6	402.7	22	299.2
7	318.8	23	7.1
8	307.7	24	0.0
9	307.3	25	0.0
10	307.0	26	0.0
11	308.0	27	0.0
12	308.7	28	0.0
13	309.3	29	75.8
14	309.2	30	302.5
15	308.5	31	314.0
16	308.2		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE November 3, 1982
COMPLETED BY R. W. Short
TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: October 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745.0	7,296.0	79,777.0
12. Number Of Hours Reactor Was Critical	596.6	7,081.5	63,320.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	591.7	7,067.5	62,157.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	616,136.8	10,044,780.1	76,746,610.2
17. Gross Electrical Energy Generated (MWH)	206,834.0	3,365,739.9	25,439,685.5
18. Net Electrical Energy Generated (MWH)	193,573.5	3,203,915.5	24,051,784.0
19. Unit Service Factor	79.4	96.9	77.9
20. Unit Availability Factor	79.4	96.9	77.9
21. Unit Capacity Factor (Using MDC Net)	54.4	91.9	65.5
22. Unit Capacity Factor (Using DER Net)	54.4	91.9	65.2
23. Unit Forced Outage Rate	20.6	3.1	3.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1983 refueling outage scheduled to commence January 3, 1983 for three months.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): None

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October 1982

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE November 3, 1982
COMPLETED BY R. W. Short
TELEPHONE (402) 536-4543

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-04	821023	F	153.3	F	1	N/A	HB	xxxxxx	Leakage from the secondary side of Steam Generator RC-2A was at a rate to warrant shutting down tne plant and effect repairs. The leakage was determined to have originated from three instrumentation handhole gaskets and several root valves. These components of RC-2A were repaired to eliminate the leakage.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending October 1982.

1. Scheduled date for next refueling shutdown. January 3, 1983

2. Scheduled date for restart following refueling. April 1, 1983

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

   a. If answer is yes, what, in general, will these be?

      A Technical Specification change

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____

   c. If no such review has taken place, when is it scheduled? _____

4. Scheduled date(s) for submitting proposed licensing action and support information. November, 1982

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:
   a) in the core 133 assemblies
   b) in the spent fuel pool 237 "
   c) spent fuel pool storage capacity 483 "
   d) planned spent fuel pool storage capacity 728 "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by J K Gasper Date November 1, 1982

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

October, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station reduced power October 6 and 7, 1982 from 85% to 65% for fuel conservation in order to meet the January 3, 1983 refueling schedule. On October 23, 1982, the plant was taken off line for unscheduled maintenance. While in cold shutdown, three handholes and several root valves on the secondary side of the steam generator were repaired due to excessive leakage. The plant was back on line October 29 and resumed 65% power October 30, 1982.

Annual licensed operator requalification training at Combustion Engineering simulator in Windsor, Connecticut was completed October 8, 1982.

Arrival of uranium hexaflouride (UF_6) was completed October 5, 1982. In all, 52 cannisters are stored on site.

The spent fuel shipping cask with spent fuel pins was shipped offsite October 1, 1982. These fuel pins are for DOE analysis of high burnup fuel.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
LER-017	During pre-operational testing of the redundant containment hydrogren monitors, a control room operator noticed that there was no valve position indication for in-board containment isolation valves, HCV-820B and 821B, associated with hydrogen monitor VA-81A. Subsequently, it was determined that the subject valves had failed open. These valve failures resulted in noncompliance with Technical Specification 2.6.(1)a. Emergency Procedure EP-25, "Loss of Containment Integrity", was immediately implemented. The redundant outboard isolation valves, HCV-820A and 821A, were verified closed and operable.
LER-018	During the re-review of all electro-pneumatic (E/P) valve positioners at the Fort Calhoun Station, the District determined that the Safety Injection (SI) leakage cooler control valves PCV-2909, 2929, 2949, and 2969 should be included as part of the safety-related electrical equipment that is environmentally qualified. This LER

A. PERFORMANCE CHARACTERISTICS (Continued)

LER-018 (Continued)

is being submitted based on the non-existence of analysis or test qualification data for these Honeywell Model No. 67400-023 positioners and the consequences of failure of these positioners. Their failure could result in the opening of the subject valves due to instrument air leakage and, thus, provide a potential path for diverting part of the required SI flow from the reactor during an accident situation. The District is reporting this potential problem as required by IE Bulletin 79-01B

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operation Incidents and are not reported elsewhere in the report:

Operation Incidents		Deficiency
01-1588	ST-ERM-1	Environmental TLD located within the DeSota township was found missing at collection time.
01-1589	ST-ESF-5	Timer for SI-2B out of specification.
01-1465	ST-CONT-3, F.1	While performing Containment Isolation Valves, leakage rate testing Type C penetrations M-2 and M-4 were found beyond maximum leakage allowed.
01-1377	ST-PORV-1	While performing ST-PORV-1, the PORV's opened when HC-102/1A was placed in "Test" position.
01-957	ST-RM-2, F.1	During the performance of ST-RM-2, F.1, Annual Calibrations were found out of specification for RM-053, 054B, 55, 56A, 56B.

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS (Continued)

Operations Incident		Deficiency
01-1594	ST-ESF-11, F.1	B/PIC-905 failed to "Reset" from tripped Condition at the appropriate setpoint.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-TSCFP-1	Technical Support Center Fire Protection System Hydrostatic Test. Test showed no leakage. This procedure did not constitute an unreviewed safety question because it only provided for a hydrotest of a portion of the fire protection system. The requirements of Technical Specification 2.19 were followed.
SP-FAUD-1	Fuel Assembly Uplift Condition Detection.
10-11-82	This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF

Phillip Avery was added to our Operations Department in the month of October.

G. TRAINING

NRC licensed personnel and license candidates attended Combustion Engineering simulator.

Non licensed operators had fire brigade and systems training.

Maintenance personnel received systems training, additional emergency response training and a safety meeting was conducted.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

NONE

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
16385	8-30-82	Fire Pump FP-1B clean heat exchanger on coolant system.	Completed.
16381	9-21-82	Replace heat exchanger regulator on Fire Pump FP-1B.	Completed.
16368	8-25-82	HCV-884A will not give closed initiation.	Adjusted limit switch.
14058	3-2-82	Fire Pump FP-1B will not hold packing.	Installed upper packing.
16678	10-1-82	DTR-111/121 ΔT for loop reading 3^{o}F low.	Replaced recorder amp.
16581	9-20-82	Charging Pump CH-1A packing leaking excessively.	Completed MP-CH-1.
16188	10-18-82	Raw Water Pump AC-10-C overhaul.	Completed per procedure AC-10-1.
16927	10-24-82	ICI Flange tighten all nuts.	Completed per procedure.

W. G. Gates
Plant Manager