

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

OCTOBER, 1982

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by John Ullrich, telephone number (815)357-6761, extension 481.

II. SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE

October 1-2 The unit started the reporting period at a load of 150 MWe. At 0129 on October 2 the load was reduced to 0 MWe and the main turbine was secured so that STP-71 "RHR Steam Condensing" could be performed. At 1405 on October 2 the main generator was synchronized to the grid and load increased to 150 MWe.

October 2-6 Load was maintained at approximately 150 MWe until 1150 on October 6 when the Unit was manually scrammed due to low reactor water level. The cause of the low water level was the loss of all the feedwater pumps due to inadvertently shorting out the low suction pressure interlock while troubleshooting. Reactor vessel water level was restored by RCIC and the motor driven feed water pump.

October 9-19 Reactor was brought critical at 2215 on October 9. At 2115 on October 10 the main generator was synchronized to the grid and load increased to 140 MWe. At 1345 on October 17, load was increased slowly until 300 MWe was reached at 2100 on October 18. At 1111 on October 19 load was reduced to 0 MWe and the main turbine was secured to repair a steam leak on the First Stage Pressure sensing line.

October 20-31 At 0345 on October 20 the main generator was synchronized to the grid and loaded to 135 MWe. By 1500 load was increased to 365 MWe. At 1600 on October 22 load was increased slowly until at 0845 on October 24 load was at 490 MWe. At 0346 on October 29 load was decreased to 450 MWe due to Condensate Polisher problems. Due to these CP problems by the end of the reporting period, load was reduced to 400 MWe.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications.

AMENDMENT 5 This Amendment consists of changes to the Technical Specifications. The changes to the Technical Specifications were as follows: (1) demonstrate the RCIC system is operable at least every 18 months, by verifying, using the test flow path, that the system is capable of delivering a flow greater than or equal to 600 gpm to the reactor vessel when steam is supplied to the turbine at a pressure of 150 ± 15 psig, and (2) allow the use of initial startup test data for determining equivalent flows in the RCIC test line and vessel injection line.

AMENDMENT 6 This Amendment consists of a change to the Technical Specifications to include the Unit 2 Service Water System Effluent Line Radiation Monitor (2D18-K608) in Tables 3.3.7.10-1 and 4.3.7.10-1 of Specification 3.3.7.10.

AMENDMENT 7 This Amendment consists of a change to paragraph 2.C(30)(o) of the Facility Operating License No. NPF-11. The change is as follows:

Upgrade Emergency Support Facilities (III.A.1.2, SER, SSER#1)

The licensee shall complete its Emergency Response Facilities as follows:

- | | | |
|-------|---------------------------------|-------------------|
| (i) | Safety Parameter Display System | December 31, 1982 |
| (ii) | Emergency Operations Facility | December 31, 1982 |
| (iii) | Technical Support Center | October 1, 1982 |

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE (CONTINUED)

AMENDMENT 8 This Amendment consists of a change to Technical Specification 6.1; Organization, Review, Investigation and Audit. This change specifies the requirements for the Operating Assistant Superintendent.

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L11690	---	OPS-VE145	Pressure Switch out-of-adjustment	Did not operate within acceptable limits	Recalibrated switch
L18616	---	APRM Channels	Channels out-of-adjustment	Showing greater than the 4% tolerance limit	Recalibrated APRM recorders
L18930	---	'O' D/G Fuel Pressure	Gauge out-of-adjustment	Pressure reading low	Recalibrated gauge
L19083	---	Div 1 Suppression Pool Temperature	Blown fuse	Thermocouple $> 5^{\circ}\text{F}$ difference from others in same well	Replaced fuse and meter movement
L19096	---	Rx Vessel Low Level Scram Switch	Bad R.H. and L.H. switch	Switch setpoint is non-repetitive	Replaced R.H. and L.H. switches
L19267	---	'C' Rx Bldg Vent Exhaust Rad Monitor	Bad G.M. tube	Detector reading about 10 times what other channels are reading	Replaced G.M. tube
L19324	---	'D' Fuel Pool Exhaust Rad Element	Bad G.M. tube	Indicates a factor of 5 over other 3 units	Replaced G.M. tube
L19439	82-133/03L-0	1C RHR SW Pump	Bad motor	Motor tripped and set off smoke detector	Replaced motor
L19507	82-114/01T-0	B RHR SW PRM Sump Pump	Blown Gasket	Pump leaks	Replaced gasket
L19540	82-115/03L-0	HPCS Testable Check Bypass Valve	Actuator spring loose	Valve stuck open	Wound spring
L19541	82-115/03L-0	HPCS Testable Check Valve	Reciprocating parts dirty	Valve stuck open	Cleaned and lubricating all reciprocating parts
L19546	---	'1A' RHR Service Water Pump	Bad Outboard bearing	Caused high vibration	Replaced outboard bearing
L19580	---	'1B' Main Steam Line Rad Monitor	Instrument out-of-tolerance	Monitor reading about 50-60% as high as other 3 monitors	Recalibrated Instrument

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L19600	---	SBGT Flow Recorder	Instrument out of calibration	Reads 200 cfm when there is no flow	Recalibrated instrument
L19617	82-117/03L-0	U-1 Div I 125 VDC Charger	Blown fuse and bad amplifier board	Charger not putting any amps out	Replaced fuse and amplifier board
L19637	---	ADS 'E' Accumulator Low Pressure	Loose switch	Control room alarm is up	Tightened switch
L19642	---	1B RHR Service Water Pump	Existing oiler too large	Oiler cannot be set to maintain proper oil level in bearing housing	Replaced oiler
L19643	---	1A RHR Service Water Pump	Existing oiler too large	Oiler cannot be set to maintain proper oil level in bearing housing	Replaced oiler
L19687	---	LPRM 3B-48-41	Bad LPRM card	LPRM indicating high	Replaced LPRM
L19708	---	'A' VE Air Condenser Compressor	Back pressure regulator out-of-adjustment	Operating at 1/2 capacity	Adjusted back pressure regulator
L19714	---	Intermediate Range Monitor	Transient times too short	When switching IRM's from range 6 to 7, get a momentary half scram	GE is investigating problem
L19770	82-125/03L-0	Div II Fuel Pool Radiation Hi-Hi Alarm	Bad K2 relay	Back panel drawers showed no trip lights or upscale indication	Replaced K2 relay
L19778	---	LPRM 08-49C	Bad LPRM card	LPRM channel failed downscale	Replaced LPRM card
L19865	82-135/03L-0	'B' Rx Bldg Vent Exhaust PRM	Bad GM tube	Monitor drifting upward	Replace GM tube
L19866	---	'O' D/G Ventilation	Blown fuse	Controller will not operate outside air damper	Replace fuse
L19874	82-134/03L-0	U-1 Rx Bldg Ventilaton D/P	Closed damper	Ventilation will not maintain Rx Bldg at less than -.25" H ₂ O	Opened damper

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L19876	---	Steam Tunnel Hi Δ T Alarm	Selector switch dirty	Hi Δ T alarm up. Indication also shows hi	Cleaned selector switch
L19974	---	Div I Post LOCA Monitor	Broken pen drive motor	Excessive noise and scribbles all over paper	Repaired pen drive motor
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IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, October 1 to October 31, 1982. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
82-111/03L-0	9/27/82	Main Steam Leak to Radwaste Reboiler
82-112/03L-0	9/29/82	Rx. Vessel Low Water Level Scram, Primary Containment Isolation
82-113/03L-0	8/16/82	D Fuel Pool Vent Rad Monitor
82-114/01T-0	10/5/82	Potential Safety Concern Identified by Independent HVAC Review
82-115/03L-0	10/5/82	Failure to Close of the HPCS Testable Check and Testable Check Bypass valves
82-116/03L-0	10/5/82	Failure of U-1 Containment Cam 1PL75J
82-117/03L-0	10/8/82	Loss of Unit 1 Division 1 125V Battery Charger
82-118/03L-0	10/8/82	Station Ventilation Stack Process Radiation Monitor
82-119/03L-0	10/10/82	"0B" VC Control Room Air Condition System
82-120/03L-0	10/10/82	"1A" OG Hydrogen Analyzer
82-121/01T-0	10/12/82	Potential Safety Concern Identified by Independent HVAC Review
82-122/03L-0	10/6/82	Failure of Lake Blowdown Flow Recorder
82-123/03L-0	10/6/82	Inoperable Mechanical Snubber
82-124/03L-0	10/16/82	Flow Recorder OFT-WL001 Inop
82-125/03L-0	10/16/82	Failure of Monitor 1D18-K615D
82-126/03L-0	10/17/82	T Factor Too Low (.92)

IV. LICENSEE EVENT REPORTS (CONTINUED)

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
82-127/03L-0	9/20/82	Degradation of Spent Resin Tank Influent Header
82-128/03L-0	10/18/82	Failure of U-2 SBT System Heaters to Energize
82-129/03L-0	10/18/82	Weld Leaking on Line 1CB30AC 3/4"
82-130/03L-0	10/19/82	Weld Leak on "1A" CD/CB Pump Vent
82-131/03L-0	10/19/82	"16A" Feedwater Heater Plug Leak
82-132/03L-0	10/4/82	"1B" RHR SW Process Rad Monitor
82-133/03L-0	9/30/82	"C" RHR SW Pump Motor Failure
82-134/03L-0	10/21/82	Rx Building Vent Δ P Inadequate
82-135/03L-0	10/21/82	Drifting of Rx Bldg Vent Rad Monitor 1B
82-136/03L-0	10/22/82	Loss of Primary Containment
82-137/03L-0	10/26/82	Trip of Recirc Pumps from Fast Speed
82-138/03L-0	10/26/82	Reactor Vessel Low Low Water Level MSIV Isolation
82-139/03L-0	10/30/82	Div II Post LOCA
82-140/03L-0	10/30/82	Pretreatment Sample Line Purging

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle 1
 DATE 11-5-82
 COMPLETED BY John Ullrich
 TELEPHONE (815)357-6761 X481

OPERATING STATUS

1. REPORTING PERIOD: October 1982 GROSS HOURS IN REPORTING PERIOD: 745
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWh): 50%* MAX. DEPEND. CAPACITY (MWe-Net): 0
 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 50%
 4. REASONS FOR RESTRICTION (IF ANY): License Condition on HVAC Review

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	662.6	1786.0	1786.0
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	611.4	991.4	991.4
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	675871	**1008516	**1008516
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	173042	225419	225419
11. NET ELECTRICAL ENERGY GENERATED (MWH)	156996	195396	195396
12. REACTOR SERVICE FACTOR	NA	NA	NA
13. REACTOR AVAILABILITY FACTOR	NA	NA	NA
14. UNIT SERVICE FACTOR	NA	NA	NA
15. UNIT AVAILABILITY FACTOR	NA	NA	NA
16. UNIT CAPACITY FACTOR (Using MOC)	NA	NA	NA
17. UNIT CAPACITY FACTOR (Using Design MWe)	NA	NA	NA
18. UNIT FORCED OUTAGE RATE	NA	NA	NA

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY		6/21/82
INITIAL ELECTRICITY		9/4/82
COMMERCIAL OPERATION	11/15/82	

** The Year To Date and Cumulative Gross Thermal Energy Generated (MWH) were incorrectly reported for the month of September 1982 due to a mathematical error. These columns should have read 332645.

* License restriction

LTP-300-7
Revision 2
November 13, 1979
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ATTACHMENT A
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373

UNIT LaSalle 1

DATE 11-5-82

COMPLETED BY John Ullrich

TELEPHONE (815)357-6761 X481

MONTH October 1982

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>133</u>
2.	<u>50</u>
3.	<u>122</u>
4.	<u>132</u>
5.	<u>137</u>
6.	<u>60</u>
7.	<u>-18</u>
8.	<u>-18</u>
9.	<u>-18</u>
10.	<u>-3</u>
11.	<u>123</u>
12.	<u>143</u>
13.	<u>132</u>
14.	<u>131</u>
15.	<u>130</u>
16.	<u>134</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>169</u>
18.	<u>208</u>
19.	<u>109</u>
20.	<u>216</u>
21.	<u>332</u>
22.	<u>373</u>
23.	<u>408</u>
24.	<u>443</u>
25.	<u>447</u>
26.	<u>423</u>
27.	<u>423</u>
28.	<u>417</u>
29.	<u>410</u>
30.	<u>397</u>
31.	<u>397</u>

ATTACHMENT B
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER 1982

DOCKET NO. 050-373

UNIT NAME LaSalle 1

DATE 11/5/82

COMPLETED BY John Ullrich

TELEPHONE (815) 357-6761

X481

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
3	10-6-82	F		105.3	A	2	Manual scram due to low Reactor water level caused by the loss of the running TDRFP.

VI. UNIQUE REPORTING REQUIREMENTS

A. Main Steam Relief Valve Operations for Unit 1

There were no main steam relief valve operations for the reporting period.

B. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period:

<u>Outage No.</u>	<u>Equipment</u>	<u>Purpose of Outage</u>
1-1624-82	"1A" RHR Service Water Pump	To replace the outboard thrust bearing
1-1651-82	"B" RHR Service Water PRM Sample Pump	To repair pump casing leak
1-1658-82	"1A" RHR Service Water Pump	To replace the oiler assembly
1-1663-82	"1B" RHR Service Water Pump	To replace the oiler
1-1671-82	"BC" RHR Water Leg Pump	To lubricate bearings

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.

VI. UNIQUE REPORTING REQUIREMENTS (CONTINUED)

E. Process Control Program

There were no changes to the Process Control Program during this reporting period.