

SOUTH CAROLINA ELECTRIC & GAS COMPANY

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O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

November 2, 1982 NOV 8 P3:01

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street, N.W.
Atlanta, Georgia 30303

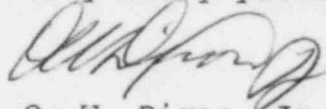
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
Thirty Day Written Report
LER 82-016 (Interim Report)

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #82-016 for Virgil C. Summer Nuclear Station. This Thirty Day Report is required by Technical Specification 6.9.1.13.(b) as a result of entry into Action Statement 16 of Technical Specification 3.3.2, "Engineered Safety Feature Actuation System Instrumentation," on October 3, 1982, and again on October 19, 1982. The attached interim report addresses both of these incidents. A final report will be submitted at a later date pending the resolution of these events.

Should there be any questions, please call us at your convenience.

Very truly yours,


O. W. Dixon, Jr.

ARK:OWD:dwf
Attachment

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Mr. James P. O'Reilly
LER No. 82-016
Page Two
November 2, 1982

DETAILED DESCRIPTION OF EVENT

On October 3, 1982, with the Plant in Mode 3, the Main Control Board Pressure Indicator (PI) 0950 was reading low, and was subsequently declared inoperable. PI-0950 is one of four such Reactor Building pressure indicators. A Maintenance Work Request (MWR) was issued for the investigation and repair of the PI. The indicator and associated power supply were tested under surveillance observation for 48 hours with no resulting abnormalities. Surveillance Test Procedure (STP) 303.001, "Reactor Building Pressure Instrument IPT-0950 Operational Test," was performed satisfactory, and the instrument was returned to operable status on October 11, 1982. On October 19, 1982, with the Plant in Mode 2, PI-0950 again failed low. An MWR was initiated for the investigation and repair of Pressure Transmitter (PT) 0950. PT-0950 is the sensing device for PI-0950. For both incidents, Action Statement 16 of Technical Specification 3.3.2 was complied with.

PROBABLE CONSEQUENCES

There was no adverse consequence as three (3) channels of Reactor Building pressure indication still remained OPERABLE per the applicable Technical Specifications.

CAUSE(S) OF THE OCCURRENCE

The cause of the occurrence is presently under investigation by the licensee.

IMMEDIATE CORRECTIVE ACTIONS TAKEN

The inoperable pressure indication train was placed in the bypassed position per Action Statement 16 of Technical Specification 3.3.2. An MWR was initiated for the investigation and repair of the PI for the October 3rd incident. The PI tested satisfactory and was declared operable per the applicable Surveillance Test Procedure. For the October 19th incident, MWR-04884 was initiated for the investigation and repair of the PT. This MWR is presently being pursued by the licensee.

ACTION TAKEN TO PREVENT RECURRENCE

Corrective action to prevent recurrence is pending the results of the aforementioned open MWR-04884.