

February 1977

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE February 29, 1980

COMPLETED BY B. Brown

TELEPHONE 309-654-2241 ext. 245

OPERATING STATUS 0000 020180
2400 022980

1. Reporting period: _____ Gross hours in reporting period: 696

2. Currently authorized power level (MWt): 2511 Max. depend. capacity
(MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>696.0</u>	<u>1440.0</u>	<u>56154.1</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>696.0</u>	<u>1440.0</u>	<u>53480.9</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>1532337</u>	<u>3279550</u>	<u>107823531</u>
10. Gross electrical energy generated (MWH)	<u>500761</u>	<u>1076887</u>	<u>34699941</u>
11. Net electrical Energy Generated	<u>477145</u>	<u>1026916</u>	<u>32442452</u>
12. Reactor service factor	<u>100.0</u>	<u>100.0</u>	<u>82.1</u>
13. Reactor availability factor	<u>100.0</u>	<u>100.0</u>	<u>87.1</u>
14. Unit service factor	<u>100.0</u>	<u>100.0</u>	<u>78.2</u>
15. Unit availability factor	<u>100.0</u>	<u>100.0</u>	<u>79.5</u>
16. Unit capacity factor (Using MDC)	<u>89.1</u>	<u>92.7</u>	<u>61.7</u>
17. Unit capacity factor (Using Des. MWe)	<u>86.9</u>	<u>90.4</u>	<u>60.1</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>8.0</u>
19. Shutdowns scheduled over next 6 months (Type, date, and duration of each):			

20. If shutdown at end of report period, estimated date of startup: NA

* The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

8211040441 800303
PDR ADOCK05000254
R PDR

1 (final)

W. C. O. S. P.

February 1977

OPERATING DATA REPORT

DOCKET NO. 50-265
 UNIT TWO
 DATE February 29, 1980
 COMPLETED BY B. Brown
 TELEPHONE 309-654-2241 ext. 245

OPERATING STATUS 0000 020180
 2400 022980

1. Reporting period: _____ Gross hours in reporting period: 696
 2. Currently authorized power level (MWt): 2511 Max. depend. capacity
 (MWe-Net): 769* Design electrical rating (MWe-Net): 789
 3. Power level to which restricted (if any) (MWe-Net): NA
 4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	0.0	0.0	53128.9
6. Reactor reserve shutdown hours	0.0	0.0	2985.8
7. Hours generator on line	0.0	0.0	50791.8
8. Unit reserve shutdown hours.	0.0	0.0	702.9
9. Gross thermal energy generated (MWH)	0	0	103016109
10. Gross electrical energy generated (MWH)	20	27	32863077
11. Net electrical Energy Generated	-13270	-27085	30715440
12. Reactor service factor	0.0	0.0	78.7
13. Reactor availability factor	0.0	0.0	83.1
14. Unit service factor	0.0	0.0	75.2
15. Unit availability factor	0.0	0.0	76.3
16. Unit capacity factor (Using MDC)	-2.5	-2.4	59.2
17. Unit capacity factor (Using Des. MWe)	-2.4	-2.4	57.7
18. Unit forced outage rate	0.0	0.0	9.2
19. Shutdowns scheduled over next 6 months (Type, date, and duration of each):			

20. If shutdown at end of report period, estimated date of startup: March 15, 1980

* The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

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W. C. O. S. A.

APPENDIX B
AVERAGE DAILY UNIT-POWER LEVEL

QTR 500-511
Revision 4
June 1976

Docket No. 50-254

Unit ONE

Date 2-29-80

Completed by B. Brown

Telephone 309-654-2241 ext.
245

MONTH February 1980

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>732.4</u>
2.	<u>442.6</u>
3.	<u>560.1</u>
4.	<u>747.2</u>
5.	<u>762.4</u>
6.	<u>760.5</u>
7.	<u>756.8</u>
8.	<u>751.7</u>
9.	<u>737.9</u>
10.	<u>710.3</u>
11.	<u>764.1</u>
12.	<u>679.9</u>
13.	<u>739.2</u>
14.	<u>734.5</u>
15.	<u>736.6</u>
16.	<u>733.5</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>708.5</u>
18.	<u>726.0</u>
19.	<u>734.1</u>
20.	<u>726.8</u>
21.	<u>724.2</u>
22.	<u>719.9</u>
23.	<u>713.7</u>
24.	<u>294.0</u>
25.	<u>399.9</u>
26.	<u>603.3</u>
27.	<u>711.7</u>
28.	<u>748.0</u>
29.	<u>721.5</u>
30.	<u></u>
31.	<u></u>

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JUN 20 1976

O.C.O.S.A.

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT-POWER LEVEL

Revision 4
June 1976

Docket No. 50-265
Unit TWO
Date 2-29-80
Completed by B. Brown
Telephone 309-654-2241 ext. 245

MONTH February 1980

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-19.2</u>
2.	<u>-19.5</u>
3.	<u>-17.5</u>
4.	<u>-19.3</u>
5.	<u>-18.4</u>
6.	<u>-8.3</u>
7.	<u>-29.3</u>
8.	<u>-17.3</u>
9.	<u>-18.8</u>
10.	<u>-19.0</u>
11.	<u>-19.0</u>
12.	<u>-18.8</u>
13.	<u>-21.3</u>
14.	<u>-21.4</u>
15.	<u>-20.5</u>
16.	<u>-18.3</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-17.0</u>
18.	<u>-18.8</u>
19.	<u>-21.5</u>
20.	<u>-20.0</u>
21.	<u>-18.5</u>
22.	<u>-19.5</u>
23.	<u>-18.1</u>
24.	<u>-18.4</u>
25.	<u>-18.4</u>
26.	<u>-19.1</u>
27.	<u>-18.3</u>
28.	<u>-19.4</u>
29.	<u>-20.1</u>
30.	<u> </u>
31.	<u> </u>

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JUN 20 1976

O.C.O.S.A.

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 5
March 1978

DOCKET NO. 50-254

UNIT NAME Quad-Cities Unit One

DATE March 3, 1980

REPORT MONTH February 1980

COMPLETED BY D. Hannum

TELEPHONE 309-654-2241 ex. 179

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NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
2	2-2-80	F	---	B	NA	NA	NA	NA	Load reduction for investigation of potential condenser tube leaks.
3	2-24-80	S	---	H	NA	NA	NA	NA	Load reduction for one-line control rod sequence exchange.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 5
March 1978

DOCKET NO. 50-265

UNIT NAME Quad-Cities Unit Two

DATE March 3, 1980

REPORT MONTH February 1980

COMPLETED BY B. Brown

TELEPHONE 309-654-2241 ext. 245

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NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
13	791125	S	696	C	1	NA	NA	NA	Unit Two Cycle 4 Refueling Outage

QUAD-CITIES REFUELING
INFORMATION REQUEST

March 1978

1. Unit: 1 Reload: 5 Cycle: 6
2. Scheduled date for next refueling shutdown: 8-31-80 (Shutdown (EOC5))
3. Scheduled date for restart following refueling: 12-7-80 (Startup BOC6)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 reloads for future cycles of Quad Cities Unit 1. The review will be conducted in June, 1980.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: June, 1980 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
New fuel designs: Retrofit 8x8 fuel 2.65 w/o% (~ 224)
7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 596
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
- b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September, 1985
(end of batch discharge capability)

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Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

Revision 1
March 1978

1. Unit: 2 Reload: 4 Cycle: 5
2. Scheduled date for next refueling shutdown: 11-4-79 (Shutdown EOC)
3. Scheduled date for restart following refueling: 1-17-80 (Startup BOC)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 Reloads for future cycles of Quad Cities Unit 2. The review will be conducted by early September, 1979.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: Early August, 1979 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
New Fuel Design: Retrofit 8x8 fuel (180)
- a) nat. U at bundle top and bottom
 - b) two larger water rods
 - c) new enrichment distribution
 - d) prepressurized

This fuel design was previously used for Quad Cities Unit 1 Cycle 5 and Dresden Unit 2 Cycle 7.

7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 492
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
 - b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March, 1986
(End of batch discharge capability)

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APR 20 1978

Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviations, which may have been used in the Monthly Report, are defined below:

CRD	-	Control Rod Drive System
SBLC	-	Stand-By Liquid Control System
MSIV	-	Main Steam Isolation Valve
RHRS	-	Residual Heat Removal System
RCIC	-	Reactor Core Isolation Cooling System
HPCI	-	High Pressure Coolant Injection System
SRM	-	Source Range Monitor
IRM	-	Intermediate Range Monitor
LPRM	-	Local Power Range Monitor
APRM	-	Average Power Range Monitor
TIP	-	Traveling Incore Probe
RBCCW	-	Reactor Building Closed Cooling Water System
TBCCW	-	Turbine Building Closed Cooling Water System
RWM	-	Rod Worth Minimizer
SBGTS	-	Stand-By Gas Treatment System
HEPA	-	High-Efficiency Particulate Filter
RPS	-	Reactor Protection System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
RBM	-	Rod Block Monitor
BWR	-	Boiling Water Reactor
ISI	-	In-Service Inspection
MPC	-	Maximum Permissible Concentration

PCI	-	Primary Containment Isolation
SDC	-	Shutdown Cooling Mode of RHRS
LLRT	-	Local Leak Rate Testing
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
R.O.	-	Reportable Occurrence
DW	-	Drywell
Rx	-	Reactor
EHC	-	Electro-Hydraulic Control System
MCPR	-	Minimum Critical Power Ratio
PC10MR	-	Preconditioning Interim Operating Management Recommendations
LER	-	Licensee Event Report
ANSI	-	American National Standards Institute
NIOSH	-	National Institute for Occupational Safety and Health
ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring