

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE May 1, 1980

COMPLETED BY B Brown

TELEPHONE 309-654-2241,
Ext 245

OPERATING STATUS

0000 040180

1. Reporting period: 2400 043080 Gross hours in reporting period: 719

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>719.0</u>	<u>2903.0</u>	<u>5761.1</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>719.0</u>	<u>2903.0</u>	<u>54943.9</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated(MWH)	<u>1652783</u>	<u>6642019</u>	<u>111186000</u>
10. Gross electrical energy generated(MWH)	<u>535624</u>	<u>2173081</u>	<u>35796135</u>
11. Net electrical energy generated(MWH)	<u>494512</u>	<u>2051141</u>	<u>33466677</u>
12. Reactor service factor	<u>100.0</u>	<u>100.0</u>	<u>82.4</u>
13. Reactor availability factor	<u>100.0</u>	<u>100.0</u>	<u>87.3</u>
14. Unit service factor	<u>100.0</u>	<u>100.0</u>	<u>78.6</u>
15. Unit availability factor	<u>100.0</u>	<u>100.0</u>	<u>79.9</u>
16. Unit capacity factor (Using MDC)	<u>87.2</u>	<u>89.6</u>	<u>60.7</u>
17. Unit capacity factor (Using Des.MWe)	<u>89.4</u>	<u>91.9</u>	<u>62.3</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>7.8</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

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PDR ADOCK 05000254
R PDR

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE May 1, 1980

COMPLETED BY B Brown

TELEPHONE 309-654-2241,
Ext 245

OPERATING STATUS

0000 040180

1. Reporting period: 2400 043080 Gross hours in reporting period: 719
2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789
3. Power level to which restricted(if any)(MWe-Net): NA
4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>177.3</u> 153.3	<u>177.3</u> 153.3	<u>68,973</u> 53282.2
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>119.7</u> 71.7	<u>119.7</u> 71.7	<u>50,911.5</u> 50863.5
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated(MWH)	<u>167344</u>	<u>167344</u>	<u>103183453</u>
10. Gross electrical energy generated(MWH)	<u>46936</u>	<u>46985</u>	<u>32910035</u>
11. Net electrical energy generated(MWH)	<u>39394</u>	<u>1728</u>	<u>30744253</u>
12. Reactor service factor	<u>21.3</u>	<u>5.3</u>	<u>77.3</u>
13. Reactor availability factor	<u>21.3</u>	<u>5.3</u>	<u>81.6</u>
14. Unit service factor	<u>10.0</u>	<u>2.5</u>	<u>73.7</u>
15. Unit availability factor	<u>10.0</u>	<u>2.5</u>	<u>74.8</u>
16. Unit capacity factor (Using MDC)	<u>6.9</u>	<u>.1</u>	<u>56.5</u>
17. Unit capacity factor (Using Des.MWe)	<u>7.1</u>	<u>.1</u>	<u>58.0</u>
18. Unit forced outage rate	<u>62.3</u>	<u>62.3</u>	<u>9.4</u>
19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):			
20. If shutdown at end of report period,estimated date of startup			<u>NA</u>

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE May 1, 1980

COMPLETED BY B Brown

TELEPHONE 309-654-2241,
Ext 245

MONTH April 1980

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>739.8</u>
2.	<u>729.0</u>
3.	<u>719.2</u>
4.	<u>737.0</u>
5.	<u>743.6</u>
6.	<u>693.0</u>
7.	<u>736.7</u>
8.	<u>724.3</u>
9.	<u>721.0</u>
10.	<u>720.0</u>
11.	<u>712.5</u>
12.	<u>720.1</u>
13.	<u>708.5</u>
14.	<u>703.4</u>
15.	<u>696.5</u>
16.	<u>697.3</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>689.5</u>
18.	<u>681.0</u>
19.	<u>673.8</u>
20.	<u>666.1</u>
21.	<u>683.6</u>
22.	<u>628.7</u>
23.	<u>651.7</u>
24.	<u>656.7</u>
25.	<u>635.8</u>
26.	<u>646.4</u>
27.	<u>601.3</u>
28.	<u>648.5</u>
29.	<u>606.3</u>
30.	<u>633.5</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE May 1, 1980

COMPLETED BY B Brown

TELEPHONE 309-654-2241,
Ext 245

MONTH April 1980

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-4.9</u>
2.	<u>-4.9</u>
3.	<u>-4.8</u>
4.	<u>-5.0</u>
5.	<u>-5.0</u>
6.	<u>-4.8</u>
7.	<u>-4.5</u>
8.	<u>-5.8</u>
9.	<u>-6.0</u>
10.	<u>-5.7</u>
11.	<u>-5.9</u>
12.	<u>-7.4</u>
13.	<u>-7.2</u>
14.	<u>-8.5</u>
15.	<u>-8.4</u>
16.	<u>-9.4</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-7.5</u>
18.	<u>-8.1</u>
19.	<u>-8.4</u>
20.	<u>-11.3</u>
21.	<u>97.0</u>
22.	<u>-11.5</u>
23.	<u>86.9</u>
24.	<u>273.9</u>
25.	<u>-12.0</u>
26.	<u>-11.5</u>
27.	<u>-11.2</u>
28.	<u>229.7</u>
29.	<u>495.3</u>
30.	<u>638.3</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 5
March 1978

DOCKET NO. 50-254

UNIT NAME Quad-Cities One

DATE May 1, 1980

REPORT MONTH APRIL 1980

COMPLETED BY T. Hafera

TELEPHONE 309-654-2241,
Ext. 176

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NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
6	800429	F	0.83	A	NA	80-12/03L	CB	PUMPXX-B	1A Recirc pump tripped on low oil pressure when 1A1 Recirc M-G Set oil pump tripped; 1B Recirc pump was reduced to 30 percent power. Oil pump was restarted and performed satisfactorily; 1A Recirc pump was restarted and performed normally.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 5
March 1978

DOCKET NO. 50-265

UNIT NAME Quad-Cities Two

DATE May 1, 1980

REPORT MONTH APRIL 1980

COMPLETED BY T. Hafera

TELEPHONE 309-654-2241,
Ext. 176

*

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
13	791125	S	481.5 463:15	C	A	NA	NA	NA	Unit Two Cycle 4 Refueling Outage
1	800420	F	39.4 14:34	A	1	80-11/03L	CC	VALVEX	Repair 3C Electromatic Relief Valve
2	800422	F	18.1 63:00	A	1	80-10/03L	SE	C	Repair pressure suppression chamber vacuum breakers

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

<u>UNIT</u>	<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO. & TYPE ACTUATIONS</u>	<u>PLANT CONDITIONS</u>	<u>DESCRIPTIONS OF EVENTS</u>
2	4-20-80	2-203-3A 2-203-3B 2-203-3C 2-203-3D 2-203-3E	1 Manual 1 Manual Failed to Open 1 Manual 1 Manual	Rx Press 400	Surveillance T.S. 4.5.D.1.b. and Acoustic Monitor Modification Test
2	4-21-80	2-203-3C	Failed to Open	Rx Press 400	Test after fail to open on 4-20-80
2	4-23-80	2-203-3C 2-203-3D	3 Manual 1 Manual	Rx Press 400	Test after fail to open on 4-20-80 & Acoustic Monitor Modification Test. (2-203-3C failed to open from control room. Actuated locally 1 time. Adjusted stroke on pilot valve. Operated 2 times from control room.)

B. Control Rod Drive Scram Timing Data For Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 psig.

RESULTS OF SCRAM TIMING MEASUREMENTS

PERFORMED ON UNIT 1 & 2 CONTROL

ROD DRIVES, FROM 1-1-80 TO 12-31-80

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				Max. Time For 90% Insertion	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5		
						7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
2-28	176	0.28	0.51	1.00	1.74	F-7 1.98 sec	Cold Scram Timing Unit 2 G-9 and M-12 were timed on March 1, 1980. N-4 was not scram timed and will be after repair of scram valve limit switches.
4-15	1	0.32	0.53	0.99	1.68	N-4 1.68	Unit 2 Rod N-4 was cold scram timed to complete all of the cold scram timing.
4-21	89	0.32	0.70	1.48	2.59	M-12 3.19	Seq B Hot Scram Timing Unit 2
4-24	88	0.31	0.69	1.46	2.56	E-10 2.95	Seq A Hot Scram Timing Unit 2
4-27	4	0.28	0.51	0.98	1.69	J-9 1.75	Cold Scram Timing rods L-12, J-9, G-8 and M-6 after drive replacement.
4-28	4	0.32	0.72	1.50	2.60	J-9 2.76	Hot Scram Timing rods L-12, J-9, G-8 and M-6 after drive replacement

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978 licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et. al., titled "Dresden, Quad-Cities, and Zion Station - NRC request for refueling information dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

REVISION 1
March 1978

1. Unit: 1 Reload: 5 Cycle: 6
2. Scheduled date for next refueling shutdown: 8-31-80 (Shutdown (EOC5))
3. Scheduled date for restart following refueling: 12-7-80 (Startup BOC6)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 reloads for future cycles of Quad Cities Unit 1. The review will be conducted in June, 1980.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: June, 1980 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
New fuel designs: Retrofit 8x8 fuel 2.65 w/o% (~ 224)
7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 596
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
- b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September, 1985
(end of batch discharge capability)

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APR 20 1978

Q.C.O.S.R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

Revision 1
March 1978

1. Unit: 2 Reload: 4 Cycle: 5
2. Scheduled date for next refueling shutdown: 11-4-79 (Shutdown EOC4)
3. Scheduled date for restart following refueling: 1-17-80 (Startup EOC5)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 loads for future cycles of Quad Cities Unit 2. The review will be conducted by early September, 1979.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: Early August, 1979 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
New Fuel Design: Retrofit 8x8 fuel (180)
- a) nat. U at bundle top and bottom
 - b) two larger water rods
 - c) new enrichment distribution
 - d) prepressurized

This fuel design was previously used for Quad Cities Unit 1 Cycle 5 and Dresden Unit 2 Cycle 7.

7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 492
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
 - b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March, 1986
(End of batch discharge capability)

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APR 20 1978

Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviation which may have been used in the Monthly Report, are defined below:

CRD	-	Control Rod Drive System
SBLC	-	Standby Liquid Control System
MSIV	-	Main Steam Isolation Valve
RHRS	-	Residual Heat Removal System
RCIC	-	Reactor Core Isolation Cooling System
HPCI	-	High Pressure Coolant Injection System
SRM	-	Source Range Monitor
IRM	-	Intermediate Range Monitor
LPRM	-	Local Power Range Monitor
APRM	-	Average Power Range Monitor
TIP	-	Traveling Incore Probe
RBCCW	-	Reactor Building Closed Cooling Water System
TBCCW	-	Turbine Building Closed Cooling Water System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
HEPA	-	High-Efficiency Particulate Filter
RPS	-	Reactor Protection System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
RBM	-	Rod Block Monitor
BWR	-	Boiling Water Reactor
ISI	-	In-Service Inspection
MPC	-	Maximum Permissible Concentration

PCI	-	Primary Containment Isolation
SDC	-	Shutdown Cooling Mode of RHRS
LLRT	-	Local Leak Rate Testing
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
R.O.	-	Reportable Occurrence
DW	-	Drywell
RX	-	Reactor
EHC	-	Electro-Hydraulic Control System
MCPR	-	Minimum Critical Power Ratio
PC10MR	-	Preconditioning Interim Operating Management Recommendations
LER	-	Licensee Event Report
ANSI	-	American National Standards Institute
NIOSH	-	National Institute for Occupational Safety and Health
ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring