

# OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE October 1 1981

COMPLETED BY Robert C Tubbs

TELEPHONE 309-654-2241X174

## OPERATING STATUS

0000 090181

1. Reporting period: 2400 093081 Gross hours in reporting period: 720
2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769\* Design electrical rating (MWe-Net): 789
3. Power level to which restricted (if any) (MWe-Net): NA
4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>712.2</u>	<u>6301.6</u>	<u>67007.9</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>708.4</u>	<u>6171.8</u>	<u>64055.6</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>1645941</u>	<u>13948598</u>	<u>130190686</u>
10. Gross electrical energy generated (MWH)	<u>534250</u>	<u>4542671</u>	<u>41921578</u>
11. Net electrical energy generated (MWH)	<u>499016</u>	<u>4223845</u>	<u>39081124</u>
12. Reactor service factor	<u>98.9</u>	<u>96.2</u>	<u>81.4</u>
13. Reactor availability factor	<u>98.9</u>	<u>96.2</u>	<u>85.6</u>
14. Unit service factor	<u>98.4</u>	<u>94.2</u>	<u>77.8</u>
15. Unit availability factor	<u>98.4</u>	<u>94.2</u>	<u>78.9</u>
16. Unit capacity factor (Using MDC)	<u>90.1</u>	<u>83.8</u>	<u>61.7</u>
17. Unit capacity factor (Using Des. MWe)	<u>87.8</u>	<u>81.7</u>	<u>60.2</u>
18. Unit forced outage rate	<u>1.6</u>	<u>2.6</u>	<u>7.3</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

\*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

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# OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE October 1 1981

COMPLETED BY Robert C Tubbs

TELEPHONE 309-654-2241X174

## OPERATING STATUS

0000 090181

1. Reporting period: 2400 093081 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769\* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>145.4</u>	<u>5881.0</u>	<u>64713.8</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>144.3</u>	<u>5854.1</u>	<u>62135.3</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>236180</u>	<u>12558041</u>	<u>127758449</u>
10. Gross electrical energy generated (MWH)	<u>71734</u>	<u>3948205</u>	<u>40669756</u>
11. Net electrical energy generated (MWH)	<u>64507</u>	<u>3739209</u>	<u>38096161</u>
12. Reactor service factor	<u>20.2</u>	<u>89.8</u>	<u>79.5</u>
13. Reactor availability factor	<u>20.2</u>	<u>89.8</u>	<u>83.2</u>
14. Unit service factor	<u>20.0</u>	<u>89.4</u>	<u>76.3</u>
15. Unit availability factor	<u>20.0</u>	<u>89.4</u>	<u>77.2</u>
16. Unit capacity factor (Using MDC)	<u>11.7</u>	<u>74.2</u>	<u>60.9</u>
17. Unit capacity factor (Using Des. MWe)	<u>11.4</u>	<u>72.3</u>	<u>59.3</u>
18. Unit forced outage rate	<u>0.0</u>	<u>.9</u>	<u>8.4</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup 12-29-81

\*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE October 1 1981

COMPLETED BY Robert C Tubbs

TELEPHONE 309-654-2241X174

MONTH September 1981

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>739.1</u>
2.	<u>732.1</u>
3.	<u>749.5</u>
4.	<u>738.4</u>
5.	<u>499.8</u>
6.	<u>560.9</u>
7.	<u>592.2</u>
8.	<u>709.9</u>
9.	<u>744.0</u>
10.	<u>743.2</u>
11.	<u>742.9</u>
12.	<u>746.0</u>
13.	<u>713.7</u>
14.	<u>737.9</u>
15.	<u>742.3</u>
16.	<u>751.0</u>

17.	<u>743.8</u>
18.	<u>748.6</u>
19.	<u>761.7</u>
20.	<u>678.2</u>
21.	<u>731.8</u>
22.	<u>754.0</u>
23.	<u>235.4</u>
24.	<u>506.8</u>
25.	<u>673.8</u>
26.	<u>765.7</u>
27.	<u>711.1</u>
28.	<u>768.4</u>
29.	<u>722.8</u>
30.	<u>748.5</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE October 1 1981

COMPLETED BY Robert C Tubbs

TELEPHONE 309-654-2241X174

MONTH September 1981

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>479.1</u>
2.	<u>471.7</u>
3.	<u>478.1</u>
4.	<u>468.4</u>
5.	<u>472.3</u>
6.	<u>430.7</u>
7.	<u>-11.3</u>
8.	<u>-9.8</u>
9.	<u>-7.3</u>
10.	<u>-6.1</u>
11.	<u>-6.0</u>
12.	<u>-6.0</u>
13.	<u>-6.0</u>
14.	<u>-6.0</u>
15.	<u>-5.8</u>
16.	<u>-5.9</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>-5.7</u>
18.	<u>-4.5</u>
19.	<u>-4.3</u>
20.	<u>-4.3</u>
21.	<u>-3.1</u>
22.	<u>-2.9</u>
23.	<u>-2.8</u>
24.	<u>-2.2</u>
25.	<u>-2.2</u>
26.	<u>-2.1</u>
27.	<u>-2.0</u>
28.	<u>-2.1</u>
29.	<u>-2.0</u>
30.	<u>-2.1</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13  
Revision 5  
March 1978

DOCKET NO. 50-254  
UNIT NAME Quad-Cities Unit One  
DATE October 1, 1981

REPORT MONTH SEPTEMBER 1981

COMPLETED BY R. C. Tubbs  
TELEPHONE 309-654-2241  
ext. 174

\*

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
81-24	810905	S	0.0	B/H	5		RB	CONROD	Reduced load for weekly Turbine tests; and to change the control rod pattern
81-25	810923	F	11.6	G	3		HC	ZZZZZZ	Reactor scram on Condenser Low Vacuum, due to improper valve line-up

APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13  
Revision 5  
March 1978

DOCKET NO. 50-265

UNIT NAME Quad-Cities Unit Two

DATE October 1, 1981

REPORT MONTH SEPTEMBER 1981

COMPLETED BY R. C. Tubbs  
TELEPHONE 309-654-2241  
ext. 174

\*

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
81-17	810907	S	575.7	C	2		RC	FUELXX	End of Cycle Five Refueling Outage

## VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

### A. MAIN STEAM RELIEF VALVE OPERATIONS

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

<u>UNIT</u>	<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO. &amp; TYPE ACTUATIONS</u>	<u>PLANT CONDITIONS</u>	<u>DESCRIPTION OF EVENTS</u>
2	9-7-81	2-203-3A	1 Manual	Rx Press	Surveillance
		2-203-3B	1 Manual	980	T.S. 4.5.D.1.b
		2-203-3C	1 Manual		
		2-203-3D	1 Manual		
		2-203-3E	1 Manual		

### B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There were no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

## VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.



QUAD-CITIES REFUELING  
INFORMATION REQUEST

QTP 300-S32  
Revision 1  
March 1978

- \*  
1. Unit: 1 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: 9-12-82 (Shutdown EOC6)
3. Scheduled date for restart following refueling: 12-5-82 (Startup BOC7)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 reloads for future cycles of Quad Cities Unit 1. The review will be conducted in August, 1982.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: August, 1982 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
New fuel designs:
7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 820
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
- b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September, 1985  
(end of batch discharge capability)

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APR 20 1978

Q. C. J. S. R.

QUAD-CITIES REFUELING  
INFORMATION REQUEST

QTP 300-S32  
Revision 1  
March 1978

- \*  
1. Unit: 2 Reload: 5 Cycle: 6  
2. Scheduled date for next refueling shutdown: 8-30-81 (Shutdown EOC5)  
3. Scheduled date for restart following refueling: 12-20-81 (Startup BOC6)  
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 Reloads for future cycles of Quad Cities Unit 2. The review will be conducted by early August, 1981.  
5. Scheduled date(s) for submitting proposed licensing action and supporting information: Early August, 1981 for 10CFR50.59 related changes ~90 days prior to shutdown.  
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
New Fuel Design: 1. Barrier Fuel  
2. Control Cell Core  
7. The number of fuel assemblies.  
a. Number of assemblies in core: 724  
b. Number of assemblies in spent fuel pool: 672  
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:  
a. Licensed storage capacity for spent fuel: 1460  
b. Planned increase in licensed storage: None  
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September, 1984  
(End of batch discharge capability)

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Q.C.O.S.R.

## VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traveling Incore Probe
TSC	-	Technical Support Center