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NJK-81-358

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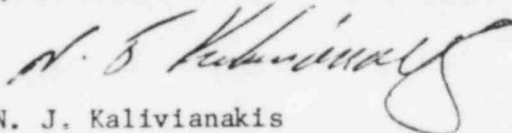
Director, Office of Management
Information and Program Control
Directorate of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Enclosed for your information is the Monthly Performance Report
covering the operation of Quad-Cities Nuclear Power Station, Units
One and Two, during the month of September 1981.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION



N. J. Kalivianakis
Station Superintendent

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Enclosure

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

SEPTEMBER 1981

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2 respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973 for Unit 2.

This report was compiled by Becky Brown and Robert Tubbs, telephone number 309-654-2241, extensions 127 and 174.

II. SUMMARY OF OPERATING EXPERIENCE

A. UNIT ONE

September 1-4: The Unit began the reporting period holding a load of 792 MWe. Power was held at that level until September 2, when Recirc flow was increased to the maximum control valve position, equal to 798 MWe. Load was held steady through September 4.

September 5-8: On September 5, load was dropped to 500 MWe, to change the rod pattern and concurrently to perform the weekly Turbine tests. Load was then increased at various rates, including a xenon soak, until 801 MWe was achieved and held at 2220 on September 8.

September 9-22: Over this 14 day period, load was held at the maximum achievable; approximately 795 MWe. The two exceptions to this were on September 13 and 20, when load was dropped briefly to 700 MWe to perform the weekly Turbine tests. Also, on September 20, load was further dropped to 550 MWe due to a minimum load condition.

September 23-26: At 0338, on September 23, the Reactor scrambled on Condenser Low Vacuum. The loss of Condenser Vacuum was due to the wrong procedure being used to transfer water from the Suppression Pool to the Condenser. The Reactor was brought critical at 1127 and the Unit put back on the system at 1515 that same day. Load was increased over the next three days until it was held at 800 MWe at 0240 on September 26.

September 27-30: On September 27, load was dropped to 700 MWe to perform the weekly Turbine tests. On the last three days of the month, load was held at 800 MWe, and the Unit ended the reporting period in that status.

B. UNIT TWO

September 1-6: Unit Two began the month holding maximum attainable load, 506 MWe, in the coastdown for the refueling outage. The Unit remained at maximum until 2000 on September 6, when load was dropped in order to commence the outage.

September 7-30: At 0020, on September 7, the Unit was tripped off-line to start the End of Cycle Five Refueling Outage. The outage is scheduled to last until December 29, 1981. Major work items to occur during the outage include: Torus Internal Modification, High and Low Pressure Turbines Overhaul, and install Hangers in the Drywell.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY
RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

On August 21, 1981, Amendments 74 and 67 were issued to DPR-29 and DPR-30, respectively. These Amendments consist of changes in the Technical Specifications for each of the two units which require the presence of a Shift Technical Advisor with specialized training in Reactor technology related to emergency operations. It also updates the shift manning chart listing the requirements for the Shift Technical Advisor.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure Changes Requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests and Experiments Requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the safety related maintenance performed on Unit One and Unit Two during the reporting period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q14383		Main Steam Line Radiation Monitors RE 1743 B,D	The B and D monitors read different than the A and C monitors.	The monitors were still functional and capable of performing their design function.	Recalibrated the B and D monitors.
Q14625		24/48 VDC Battery Chargers 1A1 and 1B2	Chargers did not function in the equalize mode.	The batteries were fully charged, if needed. The chargers were still capable of supplying DC out- put in the float mode.	Replaced the equalize mode timer.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q11262		RCIC Steam Line High Flow DP Switch 2-1360-1B	The high trip switch would not function.	Any one of the three other switches in the logic system were capable of initiating a Group V Isolation, if needed.	Replaced the faulty switch and performed a functional test.
Q14425		Main Steam Isolation Valve 2-203-2A	Failed Local Leak Rate Test due to packing leak.	The other inline MSIV had an acceptable leakage rate.	Replaced valve packing.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>UNIT ONE</u>	
	<u>Date</u>	<u>Title of Occurrence</u>
81-16/03L	9-2-81	SBGT Inoperable
<u>UNIT TWO</u>		
81-16/03L	9-6-81	HPCI Inoperable-- oil leak
81-17/03L	9-7-81	3E Electromaitc Relief Valve failed to open
81-18/03L	9-8-81	Local Leak Rate Test Failure
81-19/03L	9-29-81	RHR Service Water Vault--Penetration Failures

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions