

MONTHLY OPERATIONS SUMMARY

NOVEMBER 1981

At the beginning of the reporting period, the Oyster Creek Nuclear Generating Station was shut down to correct Second Stage Reheater Manway Steam leaks and perform maintenance on Electromatic Relief Valve (EMRV) Acoustic Monitors. The maintenance was completed and a reactor startup was performed on November 1 with the subsequent turbine loading, November 2. However, due to the unavailability of #2 Traversing Incore Probe (TIP #2), power was limited to 652 MWe. The malfunctioning TIP was partially repaired and on November 11 full power was achieved. Malfunctions in the Acoustic Valve Monitoring System for EMRV position indication on November 17 caused a power reduction to 653 MWe. On November 19, a half scram was experienced due to a failed power supply in #2 Average Power Range Monitor (APRM). The instrument was repaired. A temporary load reduction to 400 MWe was performed on November 21 in order to facilitate maintenance on A and B Condensate Pump Oil Coolers. Upon completion the plant load was increased to that level limited by the EMRV acoustic monitoring problem, approximately 650 MWe.

Extensive trouble-shooting and calibration was completed on the Safety and Relief Valve Monitoring System. There are two defective monitoring channels associated with "F" and "N" safety valves. I&C is working with Plant Engineering and will further investigate during the next plant shutdown as drywell access is required. An emergency Tech Spec change request was approved by the NRC to allow operation with degraded monitoring on the safety valves.

The following events were identified as potential Reportable Occurrences during November:

On November 16 the setpoint for Isolation Condenser Linebreak Sensor switches IB11A1 and IB11A2 were found to have settings less conservative than Technical Specification requirements.

On November 11 switches IP15 B & D for the Containment Spray System were found to have settings less conservative than Technical Specifications requirements.

On November 19 the III-F fuel on the core periphery was not monitored.

On November 21 the MAPLHGR was 103% of limit.

On November 24 and 27 snubbers failed functional testing.

On November 25 High Drywell Pressure sensing switch RV-46C was found to have a setting less conservative than Technical Specification requirements.

On November 19 V-17-30 in the Isolation Condenser System would not close on a signal from the Control Room.

OPERATING STATUS

UNIT NAME...OYSTER CREEK

DOCKET NUMBER...50-219

UTILITY DATA PREPARED BY...J.B. SKLAR 609-693-6013

REPORTING PERIOD... November 1981

LICENSED THERMAL POWER(MWT)...1930

NAMEPLATE RATING(GROSS MWE)...650

DESIGN ELECTRICAL RATING(NET MWE)...650

MAXIMUM DEPENDABLE CAPACITY(GROSS MWE)...650

MAXIMUM DEPENDABLE CAPACITY(NET MWE)...620

IF CHANGES OCCUR IN CAPACITY RATING SINCE LAST REPORT, GIVE REASON...
NONE

POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE)... 650

REASON FOR RESTRICTION, IF ANY...

ACOUSTIC MONITOR POSITION INDICATION

	MONTH	YEAR	CUMULATIVE
HOURS IN PERIOD	720.0	8015.0	104663.0
HOURS RX CRITICAL	697.8	5332.0	77762.6
RX RESERVE SHUTDOWN HRS.	0.0	0.0	468.2
HRS. GEN ON LINE	684.5	5027.7	75996.7
UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
GROSS THERMAL ENERGY	1254000.0	8056250.0	128202730.5
GROSS ELEC ENERGY	432950.0	2622460.0	43550705.0
NET ELEC ENERGY	416850.0	2501830.0	41940488.0
UT SERVICE FACTOR	95.1	62.7	72.6
UT AVAILABILITY FACTOR	95.1	62.7	72.6
UT CAPACITY FACTOR MDC	93.4	50.3	66.0
UT CAPACITY FACTOR DER	89.1	48.0	61.6
FORCED OUTAGE FACTOR	4.9	28.3	8.5

THE NEXT SCHEDULED OUTAGE IS TO BEGIN ON APRIL 16, 1982

AVERAGE DAILY POWER LEVEL

DOCKET #..... 50-219
 UNIT..... D C. #1
 REPORT DATE... November 14, 1981
 COMPILED BY... J.B. SKLAR
 TELEPHONE..... 609-693-6013

MONTH November 1981

DAY	MW	DAY	MW
1.	0.	17.	636.
2.	119.	18.	626.
3.	516.	19.	624.
4.	634.	20.	613.
5.	627.	21.	466.
6.	626.	22.	627.
7.	616.	23.	622.
8.	618.	24.	627.
9.	612.	25.	620.
10.	608.	26.	625.
11.	638.	27.	623.
12.	644.	28.	618.
13.	637.	29.	618.
14.	644.	30.	616.
15.	635.		
16.	641.		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November 1981

DOCKET NO. 50-219
 UNIT NAME Oyster Creek
 DATE December 10, 1981
 COMPLETED BY J. B. Sklar
 TELEPHONE 609-693-6013

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
21	10-30-81	F	35:26	B	1	N/A	ZZ	ZZZZZ	Shutdown to correct steam leaks from the Second Stage Reheater Manway

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

NOVEMBER SUMMARY OF QASL MECHANICAL MAINTENANCE

<u>EQUIPMENT</u>	<u>MAFUNCTION</u>	<u>CORRECTIVE ACTION</u>
CRD Accumulator 42-11 106 Valve	Packing Leak	Adjusted Packing
CRD Accumulator 38-43 107 Valve	Stem is Bad	Installed New Stem
1-2 S.W. Pump	Running Noisy	Disassembled and inspected pump Found no major problem, reassembled
V-111 CRD	Nitrogen Leak	Installed Rebuilt V-111 Valve
V-17-56	Packing Leak	Adjusted Packing
Snubber Cont. Spray 51' elevation Reactor Building	Leaking Oil	Replaced Snubber with a new one
Refueling Bridge Hoist 119' Elevation Reactor Building	Air Hose Leak on Hoist	Repaired Hose Leak
V-20-21	Packing Leak	Adjusted Packing
RBCOW SDC Valves on A&B Heat Exchanger's Disc.	Leak Through	Adjusted Valve Operators
AOG - 006B Valve	Leaking By	Disassembled valve and inspected Found dirt on seat (flushed out and cleaned).

NOVEMBER SUMMARY OF Q&SL MECHANICAL MAINTENANCE

[illegible]

NOVEMBER SUMMARY OF QASL ELECTRICAL MAINTENANCE

<u>EQUIPMENT</u>	<u>MAIFUNCTION</u>	<u>CORRECTIVE ACTION</u>
Reactor Manual Scram Button	Switch Sticking in TRIP position	Removed switch collar and inspected switch. Switch operated freely during subsequent testing.
CRD Accumulator #14-07	Repair seal-tite which separated from flexible fitting	Reconnected seal-tite to connector
Demin water and condensate storage tank-Level indication transmitter	Level transmitter freezing	Installed heaters to prevent level transmitter on demin water and condensate storage tank from freezing.
Valve V-5-106 Remote indicator on shutdown heat exchanger in control room	Remote valve indicator on V-5-106 in control room falling out of panel	Refasten remote valve indicator on panel 1F/2F in control room
Clean-up system - filter by-pass valve V-16-83	Double indication on valve V-16-83	Adjusted limit switch on valve V-16-83 for proper indication
Augmented Off Gas Building Conduit on Reactor Building wall	Refasten conduit to Reactor Building wall conduit, fasteners failed.	Re-supported conduit with five inch Hilti bolts and torqued to 35 foot pounds
Manual Rod Control - Notch over-ride switch	Switch is over-travelling	Replaced stop plate. Switch working properly now.
Clean-up System Valve V-16-14	Control Switch Handle will not spring return to normal	Switch shaft rubbing on cover plate. Adjusted switch and place back in service
Containment Spray System II Valve V-21-3	Lost indication of valve position	Adjusted primary stab at motor control center to make better contact.
Reactor Building Southeast Air Lock Door	Interlock on Door not working.	Cleaned interlock switch and door plunger. Place into service.

NOVEMBER SUMMARY OF QASL ELECTRICAL MAINTENANCE

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NOVEMBER SUMMARY OF QASL INSTRUMENT MAINTENANCE

<u>EQUIPMENT</u>	<u>MalFUNCTION</u>	<u>CORRECTIVE ACTION</u>
Source Range Monitor Channel No. 23	Surveillance Discrepancy (Front Panel Test)	Calibration Required and Completed
Intermediate Range Monitors	Surveillance Discrepancy (Front Panel Test)	Calibration Required and Completed
Control Rod No. 38-31	Select button does not latch in or light	Reinstalled 4K 38-31 Relay Retainer
Drywell Humidity Recorder	Not Printing	Replaced missing pens
Air Ejector Off Gas Radiation Monitor No. 1	Surveillance Discrepancy (Front Panel Test)	Recorder Calibration Required and Completed
No. 2 Traversing In-Core Probe	Detector mech. damaged	Replaced detector
Source Range Monitor No. 23	Period meter on 4F defective Sluggish response	Replaced meter with calibrated spare
Clean-up System	Low Pressure "Alarm" continuously in	Alarm Actuator (Switch) on Recorder defective replaced same
Area Radiation Monitor New Radwaste Building	No Power to Unit	Repaired multi-pin connector
Liquid Poison System	Local pressure gauge has cracked glass	Replaced broken glass and performed housekeeping

NOVEMBER SUMMARY OF QASL INSTRUMENT MAINTENANCE

<u>EQUIPMENT</u>	<u>MAIFUNCTION</u>	<u>CORRECTIVE ACTION</u>
Local Power Range Monitors	LPRM's indicate downscale, however no downscale light on local panel	Calibration Required and Completed
Intermediate Range Monitor	Frayed section of input cable	Removed bad section of cable and installed extension, checked satisfactory.
Average Power Range No. 2 Power Supply	Power supply failed downscale	Replaced power supply with calibrated spare
Intermediate Range Monitor No. 16	Failed downscale	Replaced connector on range switch cable
Drywell Air Temperature Recorder	Inoperative, broken drive cord	Replaced broken drive cord.

REFUELING INFORMATION -

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: July 1, 1982

Scheduled date for restart following refueling: mid-1983

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

A Tech Spec Change Request to incorporate G.E. fuel assemblies will be submitted by January 1, 1982.

Scheduled date(s) for submitting proposed licensing action and supporting information:

March 9, 1981 - Complete NEDO document #24195 (G.E. Reload Fuel Application for Oyster Creek) was submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric fuel assemblies - fuel design & performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - No major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core - 560
(b) in the spent fuel storage pool - 781

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The Spring 1987 Outage.*

*NOTE: This is for a normal refueling. Full core off-load, however can only be accommodated through about 1983 or 1984 with 1800 licensed locations.