OPERATING DATA REPORT

DOCKET NO. 50-289
DATE Sept. 15, 1981
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: August, 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.	5831.	61344.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	50.8
20. Unit Availability Factor	0.0	0.0	50.8
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	49.5
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	47.5
23. Unit Forced Outage Rate	100.0	100.0	42.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-I
DATE September 15, 1981
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

MONTH August, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
UNIT NAME TMI-1
DATE September 15, 1981
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

REPORT MONTH August, 1981

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	8/1/81	F	744	D	1				Regulatory Restraint Order

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

OPERATIONS SUMMARY

During the August report period the unit was in cold shutdown by order of the NRC until August 20, when Hot Functional Testing was commenced. All testing was performed using RCP heat for the RCS and Auxiliary Boilers to heat the secondary. Testing was performed for several reasons:

1. To permit operator training with the plant hot.
2. Perform testing to determine system and equipment integrity and ensure proper installation of restart modifications.
3. Perform many Tech Spec Surveillances that could not be performed since the last refueling (Spring 1979).

The following is a sequence of events for the testing through the end of August.

8/20 - Performed Makeup and RCS system valve lineups, filled RCS, drew bubble in pressurizer, pressurized to 50 pounds and performed 50 pound CRDm vent, performed various Tech Spec Surveillances and Testing.

8/21 - Established RCS "Daisey Chain" cleanup, changed seal injection filters, increased RCS pressure to 300 pounds, bumped RCPs and performed 300 pound CRDm vent, established RCP seals. Performed various T.S. Surveillances and Testing.

8/23 - Drew vacuum in condenser and placed secondary on cleanup. Performed T.S. Surveillances and Testing.

8/26 - "B" Feedwater string cleaned and in service. Performed T.S. Surveillances and Testing.

8/27 - RCS to 250^{o}F, 400 pounds. Performed T. S. Surveillances and Testing.

8/28 - RCS to 530^{o}F, 2155 pounds. Performed T.S. Surveillances and Testing.

8/29 - RCS to 530^{o}F, 2285 pounds for RCS leak test. Performed T.S. Surveillances and Testing.

8/30 - RCS to 530^{o}F, 2155 pounds.

Major tests performed were as follows.

Test new industrial cooler
Ventilation balance of Aux/Control Building
Leak test of various primary system check valves
Intermediate closed cooling flow balance
Non Nuclear instrumentation testing
Tsat meter calibration check
RCP Op/Flow functional testing
Source range instrument functional test
RCS pipe thermal checks
Diesel load test
PORV acoustic monitor test
Main steam safety valve testing
RCS leak rate testing
CRDm movement and drop times
Turbine roll and testing

There were no major problems identified during the testing. Chemistry, both primary and secondary came into spec. with only minor problems. There were no major system or equipment malfunctions. Worklists resulting from the heatup deal primarily with expected findings such as adjustment or replacement of valve packing. Specific problems, although minor in nature included:

1) Aux vacuum pump - damaged bearing
2) Poor new industrial cooler performance
3) Condensate booster pump seal repairs
4) Powdex vessel element replacement

Most problems identified have been repaired; the remaining are scheduled for repair.

The testing was very effective in giving new operators training and in verifying system integrity and reliability in preparation for restart.

MAJOR SAFETY RELATED MAINTENANCE

None performed during August.

REFUELING INFORMATION REQUEST

1. Name of Facility:

   Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

   Unknown

3. Scheduled date for restart following refueling:

   Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   If answer is yes, in general, what will these be?

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   If no such review has taken place, when is it scheduled?

   Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

   N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

   (a) 177

   (b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).