

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MARCH 1980

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50254 and 50-265

LICENSE NOS. DPR-29 and DPR-30

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## I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Inc. and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed-cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971 and March 21, 1972 respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2 respectively were October 18, 1971 and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973 for Unit 2.

This report was compiled by Becky Brown, Telephone number 309-654-2241, extension 245.

## II. SUMMARY OF OPERATING EXPERIENCE

### A. Unit One

March 1: Unit One began the reporting period holding load at 779 MWe. At 2200, load was reduced at a rate of 100 MWe/hr. until a load of 550 MWe was achieved at 0015 on March 2. The load was reduced to this level to perform a control rod pattern change. The weekly turbine-generator test was performed and the main condenser flow was reversed.

March 2: At 0131 Unit One load was increased at a rate of 8 MWe/hr.

March 3: At 1445 Unit One load was leveled at 820 MWe.

March 4: At 0100 Unit One load was reduced at a rate of 100 MWe/hr. until a load of 685 MWe was achieved at 0200. The load was dropped to change over condensate pumps and condensate demins. At 0216 the load was increased at a rate of 50 MWe/hr. to 750 MWe. At 0300 the rate was decreased to 25 MWe/hr. until a power level of 775 MWe was reached. At 0331 load was increased at a rate of 8 MWe/hr. until at 0755 a load of 800 MWe was reached.

March 5-9: Unit One held an average load of 803 MWe. At 2330 on March 8, load was decreased at a rate of 100 MWe/hr. to 700 MWe at which time the weekly turbine-generator test and a control rod pattern change were performed. At 0200 on March 9, load was increased at a rate of 8 MWe/hr. until 1515 when the load was leveled at 806 MWe.

March 10-11: Unit One held a load of 814 MWe until 2305 on March 10. At that time the load was dropped to 650 MWe at a rate of 100 MWe/hr to change over condensate demins. The load was then dropped to 400 MWe to change control rod pattern. The load was then increased at a rate of 50 MWe/hr. to 750 MWe, then 8 MWe/hr. to 815 MWe. At 1105, load was dropped to 725 MWe due to high differential pressure in the condensate demineralizer system.

March 12: At 0135, load was dropped at a rate of 100 MWe/hr. to 645 MWe due to high differential pressure in the condensate demineralizer system. At 1235, load was increased to 695 MWe.

March 13: At 0135, load was dropped at a rate of 100 MWe/hr. to 650 MWe to change over condensate demins. At 0430 the load was increased at a rate of 20 MWe/hr. for one hour, then 8 MWe/hr. until a load of 812 MWe was achieved.

March 14-15: Unit One held an average load of 813 MWe. At 1000 on March 15 the load was reduced at a rate of 100 MWe/hr. to 650 MWe. At this power the condenser flow was reversed, the weekly turbine test, monthly MSIV test, monthly Turbine stop valve test, and monthly turbine control valve test were performed.

March 16: At 0315, load was increased at a rate of 8 MWe/hr. until a level of 794 MWe was reached.

March 16-23: Unit One held an average load of 795 MWe. At 2230 on March 22, load was reduced at a rate of 100 MWe/hr to 650 MWe to reverse the main condenser flow and perform the weekly turbine test. At 0630 load was increased at 8 MWe/hr. to 775 MWe at 0230 on March 24.

March 24-29: Unit One held an average load of 770 MWe. At 2200 on March 28, load was decreased to 550 MWe at a rate of 100 MWe/hr., at this level a control rod pattern change was performed. At 0045 on March 29, load was dropped to 450 MWe to perform a control rod pattern change and change over condensate demins. At 0255 the load was increased at a rate of 8 MWe/hr. At 0900 this rate was reduced to 5 MWe/hr. to reverse the main condenser flow.

March 29-31: Unit One load was increased at a rate of 5 MWe/hr. until at 0000 on March 31 the rate was increased to 8 MWe/hr. At 1430 on March 31, load was leveled off at 813 MWe.

B. Unit Two

March 1-31: Unit Two remained shutdown for End of Cycle Four Refueling Outage.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specification

On March 20, 1980, Amendment No. 51 to License DPR-30 was issued for Quad-Cities Unit 2. This amendment authorizes changes to the plant Technical Specifications to support review of future reloads, under provisions of 10 CFR 50.59. It also modifies license condition 3.C. to assure a conservative MCPR operating limit during coastdown operation.

B. Facility or Procedure Changes Requiring NRC Approval

There were no facility or procedure changes requiring NRC approval.

C. Tests and Experiments Requiring NRC Approval

There were no tests or experiments performed during the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the safety related maintenance performed on Unit One and Unit Two during the reporting period. The headings indicated in this summary include Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

| W.R.<br>NUMBER | LER<br>NUMBER | COMPONENT                                  | CAUSE<br>OF<br>MALFUNCTION                              | RESULTS & EFFECTS<br>ON<br>SAFE OPERATION   | ACTION TAKEN TO<br>PREVENT REPETITION  |
|----------------|---------------|--|---|---|--|
| QO 3758        |               | 1A1 24/48V<br>battery charger              | The float-equalize<br>timer had a bad<br>potentiometer. | The float-equalize timer<br>switch did not function<br>properly. Battery sys-<br>tem was operable, as<br>was alternate charger. | The potentiometer was<br>cleaned and the charger<br>was tested.  |
| QO3722         | 80-05/03L     | 1B MSIV<br>(1-203-1B)                      | The exhaust res-<br>trictor was plugged.                | The valve would not<br>close after the control<br>switch was operated.  | The solenoid assembly,<br>pilot valve assembly, and<br>exercise valve were replaced.<br>The unit was cleaned. The<br>exhaust restrictor was<br>re-installed. The valve<br>was stroked and timed. |
| QO3985         | 80-08/03L     | 1/2 D.G.<br>(1/2-6602)                     | Fuse holder F24<br>was loose                            | The D.G. would not<br>start from the control<br>room. Unit One D.G.<br>was operable and off-site<br>lines were available.       | The fuse holder was tightened<br>and the D.G. was tested.  |
| QO 3984        |               | 1/2 D.G. Oil<br>Filter Housing             | The casing was<br>cracked.                              | The oil filter housing<br>was leaking. D.G. was<br>operable.  | The casing was repaired.   |
| QO 3615        | 80-07/03L     | Main Steam Line<br>Monitor (1-1705-<br>2B) | The monitor was<br>out of calibration.                  | The monitor was reading<br>low. Other monitors<br>were operable.  | The monitor was calibrated<br>and tested.  |

UNIT TWO MAINTENANCE SUMMARY

| W.R.<br>NUMBER | LER<br>NUMBER | COMPONENT                                      | CAUSE<br>OF<br>MALFUNCTION                                    | RESULTS & EFFECTS<br>ON<br>SAFE OPERATION                                       | ACTION TAKEN TO<br>PREVENT REPETITION                           |
|----------------|---------------|--|---|---|---|
| Q03275         |               | Solenoid valve<br>(2-1601-52B)                 | The solenoid valve<br>was worn.                               | The 2-1601-23 valve<br>would not open. Unit<br>was shutdown for refuel-<br>ing. | The solenoid valve<br>was rebuilt and stroked<br>three times.   |
| Q03705         | 79-27/03L     | RCIC Steam Supply<br>valve (M0-2-1301-<br>16). | The valve seat<br>was deformed.                               | The valve failed LLRT.<br>M0-2-1301-17 had<br>satisfactory leakage.             | The valve was replaced<br>and tested for operability.           |
| Q03908         | 80-05/03L     | Timer 10A-M2B                                  | The spring tension<br>on the clutch was<br>out of adjustment. | The timer's clutch<br>was not engaging.<br>Timers 3B and 4B<br>were available.  | The spring tension<br>was adjusted and the<br>timer was tested. |
| Q03909         | 80-08/03L     | Timer 10A-M1B                                  | The micro-switch<br>on the timer was<br>out of adjustment.    | The terminals 8 and 9<br>on the timer did not<br>pick up.                       | The micro-switch was<br>adjusted and tested.                    |
| Q02075         | 79-27/03L     | 1B MSIV<br>(2-203-1B)                          | The valve was worn.   | The valve failed LLRT.  | The valve was overhauled and<br>tested.                         |
| Q02863         | 80-01/03L     | SBLC Relief Valve<br>(RV 1105A)                | The valve was out<br>of adjustment.                           | The valve actuated at<br>too high of a pressure.                                | The valve was lapped,<br>adjusted, and tested.                  |
| Q02921         | 80-01/03L     | SBLC Relief Valve<br>(RV-1105B)                | The valve was out<br>of adjustment.                           | The valve actuated at too<br>high of a pressure.                                | The valve was adjusted and<br>tested.                           |



UNIT TWO MAINTENANCE SUMMARY

| W.R.<br>NUMBER    | LER<br>NUMBER | COMPONENT   | CAUSE<br>OF<br>MALFUNCTION                                    | RESULTS & EFFECTS<br>ON<br>SAFE OPERATION   | ACTION TAKEN TO<br>PREVENT REPETITION   |
|-------------------|---------------|---|---|---|---|
| QO 3329           |               | D.G. cooling water<br>Dezurik valve<br>(2-3907)             | A stud was broken.  | The locking mechanism<br>would not lock. D.G.<br>cooling was not<br>compromised.                                      | The stem, bearing, stud shaft<br>and sleeve were replaced. Flow<br>through the heat exchanger<br>was verified when the D.G.<br>was running. |
| QO3752            |               | Control Rod C-G.  | The velocity limit<br>er was leaking.                         | Water was leaking<br>through while changing<br>a control rod drive.   | The control rod was<br>replaced and tested.   |
| QO 3806 &<br>3807 |               | CRD P-5 and C-9   | The scram inlet<br>valve was out of<br>adjustment.            | The drive failed cold<br>scram timing. Unit<br>shutdown for refueling.  | The scram inlet valve was<br>adjusted and the drive was<br>scram timed.   |
| QO3936            |               | RHR shutdown<br>cooling valve.<br>(2-1001-50)               | The breaker was<br>burned.                                    | The valve would not<br>operate. Unit was shut-<br>down for refueling.   | The circuit was modified and<br>the valve was stroked<br>three times.   |
| QO 3992           |               | LPRM 48-090.  | The cable connector<br>was bad.                               | The LPRM was spiking<br>high. The other LPRMs<br>were operational.  | The cable connector was<br>replaced and calibration<br>was verified.  |
| QO0630            | 79-17/03L     | D.W. to Supp.<br>chamber vacuum<br>breaker (2-1601-<br>32A) | Relay CR 120HCJ02<br>in circuit for<br>2-1601-32A was<br>bad. | The vacuum breaker did<br>not give a red light<br>or alarm on Div. I.<br>The other vacuum<br>breakers were available. | The relay was replaced and<br>the circuit was tested.   |

UNIT TWO MAINTENANCE SUMMARY

| W.R.<br>NUMBER | LER<br>NUMBER | COMPONENT  | CAUSE<br>OF<br>MALFUNCTION                            | RESULTS & EFFECTS<br>ON<br>SAFE OPERATION   | ACTION TAKEN TO<br>PREVENT REPETITION  |
|----------------|---------------|--|---|---|--|
| Q01253         | 79-22/03L     | D.W. to Supp.<br>chamber vacuum<br>breaker (2-1601-<br>32B). | The Limit switch<br>was out of<br>adjustment.         | The vacuum breaker did<br>not give a full open<br>indication. The other<br>vacuum breakers were<br>available. | The limit switch was adjusted<br>and the vacuum breaker was<br>tested three times. |
| Q02090         | 79-27/03L     | F.W. Check valve<br>valve (2-220-62B)                        | The valve was<br>worn.                                | The valve failed<br>LLRT.   | The valve was rebuilt and<br>tested.   |
| Q03877         |               | 2-1601-21 valve<br>(S0-2-1601-21)                            | The valve<br>actuator was<br>worn.                    | The solenoid valve<br>was leaking. Valve<br>was still operable.   | The actuator was rebuilt<br>and the valve was tested<br>and timed.                 |
| Q0 2073        | 79-27/03L     | MSIV<br>(2-203-1A)   | The valve was<br>worn.                                | The valve failed<br>LLRT.   | The valve was overhauled<br>and re-tested.   |
| Q0 2310        |               | D.W. and particu-<br>late sample valve<br>(A0-2-8803)        | The valve was<br>worn.                                | The ASCO valve was<br>leaking. Valve<br>operability was not<br>affected.                                      | The valve was rebuilt and<br>operated and checked<br>for leaks.                    |
| Q0 3537        |               | D.G. (6600)  | The oil seal was<br>worn.                             | Oil was leaking around<br>the scavenge oil pump<br>shaft.   | The oil seal was replaced and<br>the D.G. was tested.                              |
| Q04198         | 80-07/03L     | Beam on pump<br>16 (201-2-36)                                | The beam was found<br>defective during<br>inspection. | The jet pump remained<br>intact. Other jet pumps<br>were inspected to be<br>satisfactory.                     | The beam was replaced.   |

UNIT TWO MAINTENANCE SUMMARY

| W.R.<br>NUMBER | LER<br>NUMBER | COMPONENT                 | CAUSE<br>OF<br>MALFUNCTION | RESULTS & EFFECTS<br>ON<br>SAFE OPERATION   | ACTION TAKEN TO<br>PREVENT REPETITION                 |
|----------------|---------------|---------------------------|----------------------------|---|---|
| QO 4264        | 80-09/03L     | TIP machine #2<br>(2-700) | Relay KA-477B<br>was bad.  | The machine did not<br>withdraw and isolate<br>on a Group II isolation<br>signal. | The relay was replaced<br>and the machine was tested. |

#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all license event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

| <u>Licensee Event<br/>Report Number</u> | <u>UNIT ONE</u><br><u>Date of<br/>Occurrence</u> | <u>Title of Occurrence</u>                                       |
|---|--|--|
| 80-06/03L                               | 2-22-80  | RCIC Steam Valves 1-1301-16 and 17 found in the closed position. |
| 80-07/03L                               | 3-8-80   | Steam Line 'B' Rad Monitor setpoint drift.                       |
| 80-08/03L                               | 3-8-80   | 1/2 Diesel Generator failed to start from Control Room.          |
| <u>UNIT TWO</u>                         |  |  |
| 80-05/03L                               | 3-3-80   | RHR Timer 'A' Loop 10A-M2A failed to actuate Relay K21A.         |
| 80-06/01T                               | 3-8-80   | 'A' and 'B' Core Spray Line crack indications.                   |
| 80-07/03L                               | 3-16-80  | Jet Pump #16 Beam Assembly Failure.                              |
| 80-08/03L                               | 3-5-80   | RHR Timers 'B' Loop 10A-M1B and 10A-M2B failed to operate.       |
| 80-09/03L                               | 3-22-80  | TIP Machine #2 failed to isolate.                                |

V. DATA TABULATIONS

The following data tabulations are presented in this report.

- A. Operating Data Report.
- B. Average Daily Unit Power Level.
- C. Unit Shutdowns and Power Reductions.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission.

- A. Main Steam Relief Valve Operations

There were no main steam relief valve operations during the reporting period.

- B. CONTROL ROD DRIVE SCRAM TIMING DATA

There were no control rod drive scram timing operations during the reporting period.

VIII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978 licensing memorandum (78-24) from D.E. O'Brien to C. Reed, et. al, titled, "Dresden, Quad-Cities, and Zion Station - NRC request for refueling information dated January 18, 1978.

# OPERATING DATA REPORT

DUCKET NO. 50-254

UNIT ONE

DATE April 1, 1980

COMPLETED BY Becky Brown

TELEPHONE 309-654-2241,  
Ext. 245

## OPERATING STATUS

0000 030180

1. Reporting period: 2400 033180 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769\* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

This Month    Yr. to Date    Cumulative

|   |                |                |                  |
|---|----------------|----------------|------------------|
| 5. Number of hours reactor was critical     | <u>744.0</u>   | <u>2184.0</u>  | <u>56898.1</u>   |
| 6. Reactor reserve shutdown hours           | <u>0.0</u>     | <u>0.0</u>     | <u>3421.9</u>    |
| 7. Hours generator on line                  | <u>744.0</u>   | <u>2184.0</u>  | <u>54224.9</u>   |
| 8. Unit reserve shutdown hours.             | <u>0.0</u>     | <u>0.0</u>     | <u>909.2</u>     |
| 9. Gross thermal energy generated (MWH)     | <u>1709686</u> | <u>4989236</u> | <u>109533217</u> |
| 10. Gross electrical energy generated (MWH) | <u>560570</u>  | <u>1637457</u> | <u>35260511</u>  |
| 11. Net electrical energy generated (MWH)   | <u>529713</u>  | <u>1556629</u> | <u>32972165</u>  |
| 12. Reactor service factor                  | <u>100.0</u>   | <u>100.0</u>   | <u>82.3</u>      |
| 13. Reactor availability factor             | <u>100.0</u>   | <u>100.0</u>   | <u>87.2</u>      |
| 14. Unit service factor                     | <u>100.0</u>   | <u>100.0</u>   | <u>78.4</u>      |
| 15. Unit availability factor                | <u>100.0</u>   | <u>100.0</u>   | <u>79.7</u>      |
| 16. Unit capacity factor (Using MDC)        | <u>90.2</u>    | <u>90.3</u>    | <u>60.4</u>      |
| 17. Unit capacity factor (Using Des. MWe)   | <u>92.6</u>    | <u>92.7</u>    | <u>62.0</u>      |
| 18. Unit forced outage rate                 | <u>0.0</u>     | <u>0.0</u>     | <u>7.9</u>       |

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

\*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

# OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE April 1, 1980

COMPLETED BY Becky Brown

TELEPHONE 309-654-2241,  
Ext. 245

## OPERATING STATUS

0000 030180

1. Reporting period: 2400 033180 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769\* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

|  | This Month    | Yr. to Date   | Cumulative       |
|--|---------------|---------------|------------------|
| 5. Number of hours reactor was critical  | <u>0.0</u>    | <u>0.0</u>    | <u>53128.9</u>   |
| 6. Reactor reserve shutdown hours  | <u>0.0</u>    | <u>0.0</u>    | <u>2985.8</u>    |
| 7. Hours generator on line   | <u>0.0</u>    | <u>0.0</u>    | <u>50791.8</u>   |
| 8. Unit reserve shutdown hours.  | <u>0.0</u>    | <u>0.0</u>    | <u>702.9</u>     |
| 9. Gross thermal energy generated(MWH)   | <u>0</u>      | <u>0</u>      | <u>103016109</u> |
| 10. Gross electrical energy generated(MWH)                                     | <u>22</u>     | <u>49</u>     | <u>32863099</u>  |
| 11. Net electrical energy generated(MWH)                                       | <u>-10581</u> | <u>-37666</u> | <u>30704859</u>  |
| 12. Reactor service factor   | <u>0.0</u>    | <u>0.0</u>    | <u>77.8</u>      |
| 13. Reactor availability factor  | <u>0.0</u>    | <u>0.0</u>    | <u>82.2</u>      |
| 14. Unit service factor  | <u>0.0</u>    | <u>0.0</u>    | <u>74.4</u>      |
| 15. Unit availability factor   | <u>0.0</u>    | <u>0.0</u>    | <u>75.4</u>      |
| 16. Unit capacity factor (Using MDC)   | <u>-1.8</u>   | <u>-2.2</u>   | <u>57.0</u>      |
| 17. Unit capacity factor (Using Des.MWe)                                       | <u>-1.8</u>   | <u>-2.2</u>   | <u>58.5</u>      |
| 18. Unit forced outage rate  | <u>0.0</u>    | <u>0.0</u>    | <u>9.2</u>       |
| 19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each): |               |               |                  |
| 20. If shutdown at end of report period, estimated date of startup             |               |               | <u>4-21-80</u>   |

\*The MDC may be lower than 769 MWe during periods of high ambient temperature, due to the thermal performance of the spray canal.



APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE April 1, 1980

COMPLETED BY Becky Brown

TELEPHONE 309-654-2241  
Ext. 245

MONTH March 1980

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. 719.3  
2. 597.6  
3. 737.5  
4. 755.5  
5. 756.5  
6. 765.4  
7. 773.9  
8. 741.5  
9. 746.7  
10. 755.0  
11. 693.0  
12. 655.5  
13. 703.8  
14. 766.6  
15. 753.3  
16. 711.3

17. 775.6  
18. 746.5  
19. 737.8  
20. 731.9  
21. 728.6  
22. 716.9  
23. 644.8  
24. 732.6  
25. 703.3  
26. 728.5  
27. 705.8  
28. 689.9  
29. 483.3  
30. 594.8  
31. 718.7

**INSTRUCTIONS**

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.



APPENDIX B  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE April 1, 1980

COMPLETED BY Becky Brown

TELEPHONE 309-654-2241,  
Ext. 245

MONTH March 1980

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. -17.4  
2. -20.1  
3. -21.1  
4. -19.2  
5. -19.3  
6. -21.8  
7. -21.5  
8. -20.1  
9. -21.4  
10. -20.0  
11. -22.3  
12. -19.9  
13. -21.4  
14. -22.7  
15. -22.0  
16. -19.6

17. -24.0  
18. -19.5  
19. -12.3  
20. -5.2  
21. -3.0  
22. -3.3  
23. -4.1  
24. -4.4  
25. -5.0  
26. -4.6  
27. -5.0  
28. -5.1  
29. -5.3  
30. -5.3  
31. -5.1

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13  
Revision 5  
March 1978

DOCKET NO. 50-254

UNIT NAME Quad-Cities One

COMPLETED BY Thomas Hafera

DATE April 1, 1980

REPORT MONTH March 1980

TELEPHONE 309-654-2241

\*

| NO. | DATE   | TYPE<br>F OR S | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING<br>DOWN REACTOR | LICENSEE<br>EVENT<br>REPORT NO. | SYSTEM<br>CODE | COMPONENT<br>CODE | CORRECTIVE ACTIONS/COMMENTS   |
|-----|--------|----------------|---------------------|--------|---------------------------------------|---------------------------------|----------------|-------------------|---|
| 4   | 800302 | S              | NA                  | B      | NA                                    | NA                              | NA             | NA                | Unit one load was reduced for a control rod pattern change.   |
| 5   | 800392 | S              | NA                  | B      | NA                                    | NA                              | NA             | NA                | Unit one load was reduced for a control rod pattern change and to change over condensate demineralizers |

APPENDIX D  
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-513  
Revision 5  
March 1978

DOCKET NO. 50-265

UNIT NAME Quad-Cities Two

COMPLETED BY Becky Brown

DATE April 1, 1980

REPORT MONTH March 1980

TELEPHONE 309-654-2241

\*

| NO. | DATE   | TYPE<br>F OR S | DURATION<br>(HOURS) | REASON | METHOD OF<br>SHUTTING<br>DOWN REACTOR | LICENSEE<br>EVENT<br>REPORT NO. | SYSTEM<br>CODE | COMPONENT<br>CODE | CORRECTIVE ACTIONS/COMMENTS       |
|-----|--------|----------------|---------------------|--------|---------------------------------------|---------------------------------|----------------|-------------------|-----------------------------------|
| 13  | 791125 | S              | 744                 | C      | 1                                     | NA                              | NA             | NA                | Unit Two Cycle 4 Refueling Outage |

QUAD-CITIES REFUELING  
INFORMATION REQUEST

Revision 1  
March 1973

1. Unit: 1 Reload: 5 Cycle: 6
2. Scheduled date for next refueling shutdown: 8-31-80 (Shutdown (EOC5))
3. Scheduled date for restart following refueling: 12-7-80 (Startup BOC6)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 reloads for future cycles of Quad Cities Unit 1. The review will be conducted in June, 1980.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: June, 1980 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
New fuel designs: Retrofit 8x8 fuel 2.65 w/o% (~ 224)
7. The number of fuel assemblies. 724  
a. Number of assemblies in core: 596  
b. Number of assemblies in spent fuel pool: 596
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:  
a. Licensed storage capacity for spent fuel: 1460  
b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September, 1985  
(end of batch discharge capability)

APPROVED

APR 20 1973

Q.C.O.S.R.

QUAD-CITIES REFUELING  
INFORMATION REQUEST

QTP 300-532  
Revision 1  
March 1978

1. Unit: 2 Reload: 4 Cycle: 5
2. Scheduled date for next refueling shutdown: 11-4-79 (Shutdown EOC4)
3. Scheduled date for restart following refueling: 1-17-80 (Startup BOC5)
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: No, Plan 10CFR50.59 Reloads for future cycles of Quad Cities Unit 2. The review will be conducted by early September, 1979.
5. Scheduled date(s) for submitting proposed licensing action and supporting information: Early August, 1979 for 10CFR50.59 related changes ~ 90 days prior to shutdown.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:  
New Fuel Design: Retrofit 8x8 fuel (180)
- a) nat. U at bundle top and bottom
  - b) two larger water rods
  - c) new enrichment distribution
  - d) prepressurized

This fuel design was previously used for Quad Cities Unit 1 Cycle 5 and Dresden Unit 2 Cycle 7.

7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
  - b. Number of assemblies in spent fuel pool: 492
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 1460
  - b. Planned increase in licensed storage: None
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March, 1986  
(End of batch discharge capability)

APPROVED

APR 20 1978

Q. C. O. S. R.

## VIII. GLOSSARY

The following abbreviations, which may have been used in the Monthly Report, are defined below:

|        |   |   |
|--------|---|---|
| CRD    | - | Control Rod Drive System                      |
| SBLC   | - | Stand-By Liquid Control System                |
| MSIV   | - | Main Steam Isolation Valve                    |
| RHRS   | - | Residual Heat Removal System                  |
| RCIC   | - | Reactor Core Isolation Cooling System         |
| HPCI   | - | High Pressure Coolant Injection System        |
| SRM    | - | Source Range Monitor                          |
| IRM    | - | Intermediate Range Monitor                    |
| LPRM   | - | Local Power Range Monitor                     |
| APRM   | - | Average Power Range Monitor                   |
| TIP    | - | Traveling Incore Probe                        |
| RBCCW  | - | Reactor Building Closed Cooling Water System  |
| TBCCW  | - | Turbine Building Closed Cooling Water System  |
| RWM    | - | Rod Worth Minimizer                           |
| SBGTS  | - | Stand-By Gas Treatment System                 |
| HEPA   | - | High-Efficiency Particulate Filter            |
| RPS    | - | Reactor Protection System                     |
| IPCLRT | - | Integrated Primary Containment Leak Rate Test |
| LPCI   | - | Low Pressure Coolant Injection Mode of RHRS   |
| RBM    | - | Rod Block Monitor                             |
| BWR    | - | Boiling Water Reactor                         |
| ISI    | - | In-Service Inspection                         |
| MPC    | - | Maximum Permissible Concentration             |

|          |   |  |
|----------|---|--|
| PCI      | - | Primary Containment Isolation  |
| SDC      | - | Shutdown Cooling Mode of RHRS  |
| LLRT     | - | Local Leak Rate Testing  |
| MAPLHGR  | - | Maximum Average Planar Linear Heat Generation Rate                                 |
| R.O.     | - | Reportable Occurrence  |
| DW       | - | Drywell  |
| Rx       | - | Reactor  |
| EHC      | - | Electro-Hydraulic Control System   |
| MCPR     | - | Minimum Critical Power Ratio   |
| PCOMR    | - | Preconditioning Interim Operating Management<br>Recommendations                    |
| LER      | - | Licensee Event Report  |
| ANSI     | - | American National Standards Institute  |
| NIOSH    | - | National Institute for Occupational Safety and<br>Health                           |
| ACAD/CAM | - | Atmospheric Containment Atmospheric Dilution/Containment<br>Atmospheric Monitoring |