

**Florida
Power**
CORPORATION

November 1, 1982
#3F-1182-01
File: 3-O-26

Mr. Darrell G. Eisenhut
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
NUREG-0737, Item II.D.1
Safety and Relief Valve Testing Program

Dear Mr. Eisenhut:

Pursuant to our letter of September 17, 1982, Florida Power Corporation (FPC) herewith provides further information on to NUREG-0737, Item II.D.1. Specifically, we will address areas which we committed to provide information in our June 30, 1982 letter (#3F-0682-23). These areas are detailed as follows:

- 1) Safety Valve Ring Settings - During a forced outage which began October 14, 1982, the Crystal River Unit 3 (CR-3) Pressurizer Safety Valves were replaced with valves that have ring settings installed which are consistent with the EPRI PWR Safety and Relief Valve Test Program results. In addition, FPC is pursuing, in conjunction with the Babcock & Wilcox (B&W) Owners Group, a fine-tuning of these ring settings based on plant specific conditions. This effort is anticipated to be complete by March 31, 1983, and, if deemed necessary to achieve the most optimum valve performance, ring settings determined through this program will be installed in the Pressurizer Safety Valves on CR-3 during our upcoming refueling outage.
- 2) Safety Valve Backpressure - FPC is in the process of concluding a plant specific evaluation of the CR-3 Pressurizer Safety Valve discharge piping backpressure. Since the EPRI testing program provided data which showed that safety valve performance can be affected by backpressure, this evaluation is necessary to assure proper valve performance. Preliminary indications show that the CR-3 discharge piping backpressure is within the EPRI test data and since the installed ring settings which are consistent with the EPRI data, proper valve operation is assured. If final data shows that our conclusion based on preliminary data is in error, we will inform you.

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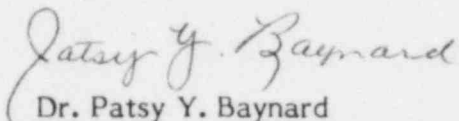
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Mr. Darrell Eisenhut
#3F-1182-01
Page 2

- 3) Safety Valve Blowdown - B&W, under the direction of the B&W Owners Group, has determined that Pressurizer Safety Valve blowdown up to 20% can be safely accommodated by the B&W Nuclear Steam Supply System. This value is in excess of the maximum seen during the EPRI test program.

From the above and our aforementioned June 30, 1982 letter, it is concluded that the CR-3 Pressurizer Safety and Relief Valves will operate as intended under all postulated transient events. Therefore, NUREG-0737, Item II.D.1 is complete.

Very truly yours,



Dr. Patsy Y. Baynard
Assistant to Vice President
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AEF/myf

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