

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 3 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
 7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 58

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 4 9 7 1 2 0 1 8 1 8 1 2 0 7 8 1 9  
 7 8 9 60 61 COCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During unit startup Rx pressure was being maintained at about 50 psig in preparation for  
 0 3 HPCI system operability test. When Rx pressure decreased to 32 psig, rod N-9 was notched  
 0 4 from position 02 to 04. Unexpectedly, the rod went to position 06 and less than a 5  
 0 5 second period resulted (T.S. 6.6.B.1.d). The reactor scrambled due to IRM H1 H1. There  
 0 6 was no effect on public health or safety because all Rx protective systems functioned  
 0 7 as designed. Previous occurrence: R.O. 50-237-76-74.

0 8  
 7 8 9  
 0 9 SYSTEM CODE R B 11 CAUSE CODE E 12 CAUSE SUBCODE X 13 COMPONENT CODE C R D R V E 14 CCMP SUBCODE Z 15 VALVE SUBCODE Z 16  
 7 8 9 10 11 12 13 14 15 16  
 17 LER/RO REPORT NUMBER 8 1 21 22 EVENT YEAR 23 SEQUENTIAL REPORT NO. 0 4 0 24 25 OCCURRENCE CODE 0 1 26 27 REPORT TYPE I 28 29 REVISION NO. 0 30 31  
 18 ACTION TAKEN X 19 FUTURE ACTION X 20 EFFECT ON PLANT A 21 SHUTDOWN METHOD C 22 HOURS 0 0 4 8 23 ATTACHMENT SUBMITTED X 24 APPD-4 FORM SUB Y 25 PRIME COMP. SUPPLIER N 26 COMPONENT MANUFACTURER G U 27 28  
 33 34 35 36 37 38 39 40 41 42 43 44 45

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of the unexpected rod movement from position 02 to 06 is unknown. The single  
 1 1 notch control switch operation and proper latching of the rod (at various positions)  
 1 2 were subsequently verified with no difficulties identified. Work request has been  
 1 3 issued to check single notch control switch during the next refueling outage. On-site  
 1 4 review conducted to ensure no safety limits were exceeded prior to unit startup.

1 5 FACILITY STATUS C 28 % POWER 0 0 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Observation 32  
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
 1 6 ACTIVITY CONTENT Z 33 RELEASED OF RELEASE Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36  
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
 1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39  
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
 1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41  
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
 1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43  
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
 2 0 PUBLICITY DESCRIPTION N/A 45  
 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

NRC USE ONLY

NAME OF PREPARER B. A. Schroeder

PHONE 815-942-2920 X-516

ATTACHMENT TO LICENSEE EVENT REPORT 81-40/01T-0

COMMONWEALTH EDISON COMPANY (CWE)

DRESDEN UNIT 3 (ILDRS-3)

DOCKET # 050-249

During unit startup, Rx pressure was being maintained at about 50 psig in preparation for HPCI system operability test. When Rx pressure decreased to about 32 psig, rod N-9 was notched from position 02 to 04. Unexpectedly, the rod went from position 02 to 06. The rod notch override switch was not being used at the time and rod withdrawal was in accordance with approved station procedures and control rod withdrawal sequences. This caused a Rx scram (IRM Hi-Hi trips). A review of SPM and IRM recorders indicated that a less than a five second period (T.S.6.6.B.1.d) occurred prior to the reactor scram.

The cause of the unexpected rod movement from 02 to 06 is unknown. The single notch control switch operation and proper latching of the rod (at various positions) were subsequently verified with no difficulties identified. The control rod timing was checked and found to be slightly fast, but within Tach. Spec. limits. The timing was adjusted and rechecked. The CRD drive pressure was normal. A work request has been issued to check the timing and contact condition of the single notch control switch during the upcoming refueling outage.

The results of the on-site investigation and review by the Nuclear Fuel Services Department found that no safety limits were exceeded. There was no effect on public health or safety because all reactor protective systems functioned as designed. If the control switch investigation proves inconclusive, the CRD will be replaced prior to startup at BOC-8. No further action deemed necessary.



Commonwealth Edison

## DEVIATION REPORT

DVR NO.

STA 12 UNIT 3 YEAR 81 NO 80

PART 1 TITLE OF DEVIATION  
Rx Scram - IRM H1 H1

OCCURRED  
12-1-81 1050  
DATE TIME

SYSTEM AFFECTED 500 PLANT STATUS AT TIME OF EVENT  
RPS MODE Startup, PWR(MWT) .1, LOAD(MWE) 0 TESTING  
YES NO

DESCRIPTION OF EVENT  
While attempting to maintain Rx critical and Rx pressure less than 50 psig, due to HPCI being inop., inserted rod N-9 from position 04 to 02. Rx went subcritical and pressure eventually decreased to 32 psig. Attempted to withdraw rod N-9 with single notch control switch from position 02 to 04 but rod moved out to position 06 and Rx scrambled due to IRM H1 H1. Preliminary investigation by nuclear engineers determined Rx period to be less than 5 seconds.

10 CFR 50.72 NRC RED PHONE NOTIFICATION MADE 1100 hrs. YES NO

EQUIPMENT FAILURE  
YES NO WORK REQUEST NO. RESPONSIBLE SUPERVISOR James R. Duxsey DATE 12-1-81

PART 2 OPERATING ENGINEER'S COMMENTS  
An immediate review of the SRM and IRM recorder traces indicated that a period of less than 5 seconds occurred prior to the Rx scram. NFS and the nuclear engineers are investigating. An onsite review will be performed prior to Rx startup.

☐ EVENT OF PUBLIC INTEREST  
☐ TECH. SPEC. VIOLATION  
☐ NON REPORTABLE OCCURRENCE  
☒ 14 DAY REPORTABLE/T.S. 6.6.B.1.d  
☐ 30 DAY REPORTABLE/T.S.  
☐ ANNUAL/SPEC. REPORT REQ'D

☒ 24-HOUR NRC NOTIFICATION REQ'D  
TELEPH Tom Tongue 12-1-81 1115  
REGION III  
TELEGM/TELECOPY Keppler 12-1-81 1455  
REGION III

☐ CECO CORPORATE NOTIFICATION MADE  
IF ABOVE NOTIFICATION IS PER 10CFR21  
☐ 5-DAY WRITTEN REPORT REQ'D PER 10CFR21

A.L.R. #  
L.E.R. # 81-40/011-0

TELEPH F. Palmer 12-1-81 1210  
CECO CORPORATE OFFICER DATE TIME

PRELIMINARY REPORT  
COMPLETED AND REVIEWED Michael Wright 12-1-81  
OPERATING ENGINEER DATE

INVESTIGATED REPORT & RESOLUTION  
ACCEPTED BY STATION REVIEW Jack Berumen 12/15/81 John M. Palmer 12/15/81

RESOLUTION APPROVED AND  
AUTHORIZED FOR DISTRIBUTION  
88-5176 (FORM 19-81-1) 10-81  
STATION SUPERINTENDENT 12-15-81  
DATE

**END**