

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	B	R	F	1	2	0	0	-	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5			
7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE					30	57 CAT 58			5

CON'T

REPORT SOURCE 0 1 6 0 5 0 0 0 2 5 9 7 0 3 2 0 8 2 8 0 9 2 7 8 2 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal operation, unidentified drywell leakage as determined by flow  
0 3 | recorder and flow integrator suddenly increased and exceeded limits specified  
0 4 | in T.S. 3.6.C.1. In accordance with T.S. 3.6.C.3 an orderly shutdown was  
0 5 | initiated and the reactor was in cold shutdown condition within 24 hours. There  
0 6 | was no danger to the health or safety of the public. The leakage never  
0 7 | approached normal makeup capacity. All emergency systems were available and  
0 8 | operable.

SYSTEM CODE C 9		CAUSE CODE B		CAUSE SUBCODE A		COMPONENT CODE P I P E X X				COMP. SUBCODE A		VALVE SUBCODE Z	
7	8	9	10	11	12	13	14	15	16	17	18	19	20
LER/RO REPORT NUMBER 8 2		EVENT YEAR 8 2		SEQUENTIAL REPORT NO. 0 2 0		OCCURRENCE CODE 0 3		REPORT TYPE X		REVISION NO. 2			
21	22	23	24	25	26	27	28	29	30	31	32		
ACTION TAKEN F X		FUTURE ACTION X		EFFECT ON PLANT A		SHUTDOWN METHOD B		HOURS 0 0 7 5		ATTACHMENT SUBMITTED Y			
33	34	35	36	37	38	39	40	41	42	43	44		
NPRD-4 FORM SUB. N		PRIME COMP. SUPPLIER A		COMPONENT MANUFACTURER Z 9 9 9									
45	46	47	48	49	50								

### CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

1 0 Leakage resulted from a vibration-induced fatigue crack in a 3/4-inch reactor water  
1 1 cleanup system test connection. Inadequate bracing of the test connection line  
1 2 allowed high-frequency vibration. The line was rerouted to decrease susceptibility  
1 3 to vibration. Inspection was completed for similar conditions on unit 3. Unit 2  
1 4 will be inspected during the cycle 4 refueling outage.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
1	5	E	28	0	9	5	29	NA	30	A	31	Operator observation	32
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE							
1	6	Z	33	Z	34	NA	35	NA	36				
PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION							
1	7	0	0	0	37	Z	38	NA	39				
PERSONNEL INJURIES		NUMBER		DESCRIPTION									
1	8	0	0	0	40			NA	41				
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION									
1	9	Z	42			8209300291	820927	PDR	ADOCK				
PUBLICITY		ISSUED		DESCRIPTION									
2	0	Y	44			S	PDR	TVA press release on	3/20/82				

NAME OF PREPARER William A. Roberts

PHONE: (205) 729-0788

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 82020 R2 Technical Specification Involved 3.6.C.1 & 3.6.C.3

Reported Under Technical Specification 6.7.2.b.(2)\* Date Due NRC \_\_\_\_\_

Event Narrative:

Unit 1 - 1,045 MWe (steady state)  
Unit 2 - 1,085 MWe (preconditioning up to full load)  
Unit 3 - 0 MWe (refuel outage)

During normal operation, unidentified drywell leakage as determined by the floor drain sump pump flow from FR-77-6 and flow integrator FQ-77-6 suddenly increased and exceeded limits specified in Technical Specification 3.6.C.1. In accordance with Technical Specification 3.6.C.3, an orderly shutdown was initiated and the reactor was in cold shutdown within 24 hours. There was no danger to the health and safety of the public. All emergency systems were available and operable. An inspection identified the leakage as being from a crack in a 3/4-inch test connection which ties into the 6-inch pipe adjacent to reactor water cleanup system valve FCV-69-1. This test line and its associated manual isolation valves (69-583 and 69-584) were inadequately supported which led to high-frequency fatigue and subsequent cracking of the line. Repair was made by removing a 90-degree elbow and shortening the line thus orientating the test valves in vertical plane perpendicular to the 6-inch line instead of being orientated in a horizontal plane parallel to the 6-inch pipe. The crack in the 3/4-inch pipe occurred above a socket weld and extended around approximately 60 percent of the circumference. The broken test line was repaired.

(see attached page)

\* Previous Similar Events:

NONE

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP

LER SUPPLEMENTAL INFORMATION  
BFRO-50-259/82020 R2

The average flow rate of water from the cracked line was approximately 12 gallons per minute. The total amount of water lost during the event was approximately 17,200 gallons. These values were derived from the actual flow rate as recorded on strip chart FR-77-6 for a 24-hour period on March 20, 1982.

During the inspection to locate the leak, it was noted that this vent connection had previously been provided with a vibration support. Support requirements for this vent connection were not shown on TVA drawings. The support had apparently been removed at some time for maintenance or modification work. Reinstallation had not been performed because vibration support requirements were not shown on design drawings. The fatigue crack occurred because the support had not been reinstalled on the line.

An evaluation of any suspect units 1 and 2 safety-related piping system will be conducted to verify that similar problems do not exist on other test, vent, or drain connections. If any discrepancies are found, appropriate corrective action will be taken. This inspection will be performed during the unit 1, cycle 5 and unit 2 cycle 4 refueling outages presently scheduled for completion July 17, 1983, and January 30, 1983, respectively.