

## (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE					
S	H	E	B	V	A	L	V	E	X	E	D						
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.									
8	2	0	8	1	0	3	L	0									
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
X	Z	Z	Z	Z	0	0	0	0	Y	N	A	R	3	4	0		

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
G	000	NA	A Alarm	
ACTIVITY CONTENT RELEASED OF RELEASE	AMOUNT OF ACTIVITY			LOCATION OF RELEASE
Z	Z	NA	NA	

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	Z	38	NA	

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	8	0	0	40	NA

1		2		3		4		5		6		7		8		9		10		11		12	
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PUBLICITY  
 ISSUED DESCRIPTION (45)  
 2 0 N (44) NA  
 8209300162 820921  
 PDR ADOCK 05000366  
 S PDR  
 NRC USE ONLY

PHONE: (912) 367-7851

LER No.: 50-366/1982-081  
Licensee: Georgia Power Company  
Facility: Edwin I. Hatch  
Docket #: 50-366

Narrative Report  
for LER 50-366/1982-081

On August 25, 1982, with the unit at hot shutdown following a reactor scram, the suppression chamber (Torus) high water level alarm was received in the control room. One indicator showed a water level of approximately 12 feet, 7 inches. Tech. Specs. section 3.6.2.1.a. states that the suppression chamber (Torus) shall be operable with a water volume equivalent to a water level between 12 feet 2 inches and 12 feet 6 inches. The limiting condition for operation (LCO) of Tech. Specs. section 3.6.2.1., Action item a. was complied with since the unit was already at hot shutdown and was at cold shutdown within 24 hours of this event. The health and safety of the public were not affected. This event is non-repetitive.

The cause of this event was a reactor scram and group I isolation that occurred as a result of a Main Steam Isolation Valve (MSIV) failing and going closed. After the scram, the High Pressure Coolant Injection (HPCI) system and the Reactor Core Isolation Cooling (RCIC) system started to help control reactor pressure and maintain reactor water level. Steam Relief Valves "A" and "D" (opened to relieve reactor pressure) discharged to the Torus and caused the Torus water level to rise above the Tech. Specs. limit.

The Torus water level was returned to Tech. Specs. limits before the unit was taken from cold shutdown. The MSIV whose failure initiated this event was repaired and returned to service.