

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket No. 50-282
50-306

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NOS. DPR-42 & DPR-60

(License Amendment Request Dated September 14, 1982)

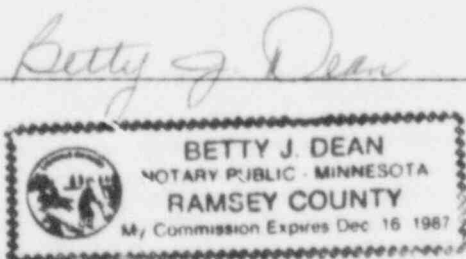
Northern States Power Company, a Minnesota corporation, request authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This letter contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By *D M Musolf*
D M Musolf
Manager of Nuclear Support Services

On this 14th day of September, 1982 before me a notary public in and for said County, personally appeared D M Musolf, Manager of Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.



8209210198

EXHIBIT A

Prairie Island Nuclear Generating Plant

License Amendment Request Dated September 14, 1982

Proposed Changes to the Technical Specifications
Appendix A of , ating Licenses DPR-42 and DPR-60

Pursuant to 10 CFR 50.59 and 50.90, the holders of operating licenses DPR-42 and DPR-60 hereby propose the following changes to Appendix A, Technical Specifications.

1. Specification: Inservice Inspection Requirements - Table TS 4.2-1

PROPOSED CHANGE

- a. Add magnetic particle (M.T.) as a method of examination to Table TS.4.2-1 item 1 for inspecting pump flywheels.
- b. Update the notes on Table TS.4.2-1 to indicate the examinations are being performed per IWA-2220 and IWA 2230.

REASON FOR CHANGE

The addition permits Prairie Island to use either magnetic particle or liquid penetrant (P.T.) methods for inspecting the pump flywheel. The remainder of the change updates the Technical Specifications to show the examinations are being done per IWA section of ASME code.

SAFETY EVALUATION

No safety evaluation is required. M.T. is an acceptable method to use for examination of ferrous material.

2. Specification: Steam Generator Tube Surveillance TS 4.12

2.1 Change to page TS.4.12-2

PROPOSED CHANGE

Make the changes to Section 4.12 shown in Exhibit B, pages 4.12-2 and 4.12-2A.

REASON FOR CHANGE

These changes allow the inspection to concentrate on a specific problem in the steam generator while meeting the C-3 criteria. The exceptions would be valid only if no other problems were identified by previous inspections.

SAFETY EVALUATION

Plant safety is not decreased by this change because the inspection would be 100% of the specific problem area.

2.2 Change to Table TS 4.12-1

PROPOSED CHANGE

Change the footnote of Table TS.4.12-1 to read:

S=3% when two steam generators are inspected during that outage.

S=6% when one steam generator is inspected during that outage.

REASON FOR CHANGE

This change is requested to minimize confusion when interpreting the Technical Specifications.

SAFETY EVALUATIONS

Safety evaluation is not required since this is not a change in the intent of the Technical Specification.

3. Specifications: Typographical Corrections to TS.3.1, TS.4.5, TS.5.6 TS.6.1

PROPOSED CHANGE

- a. Remove the redundant reference on page TS.3.1-3 as noted in Exhibit B. Reference on page TS.3.1-3A.
- b. Correct typo in TS.4.5.B.1.a by changing the word "heat" to "head".
- c. Remove the redundant reference "(1) FSAR Section 9" which appears on page TS.5.6-2.
- d. Correct Typo in TS 6.1.A.5 by changing "of Senior Reactor Operator" to "or Senior Reactor Operator".
- e. Correct typo on page TS.3.1-3 in the word maximum.

REASON FOR CHANGE

Correct typographical errors

SAFETY EVALUATION

None is required.

4. Specification: TS 6.0 Administrative Controls

4.1 Update Organizational Titles

PROPOSED CHANGE

Update the NSP Corporate Organization chart Figure TS.6.1-1 as noted in Exhibit B and the titles on pages TS 6.2-1, TS 6.2-3 and TS 6.2-5. Replace Figure TS.6.1-2 with the redrawn figure.

REASON FOR CHANGE

Recent organizational changes require an update.

SAFETY EVALUATION

None is required.

4.2 Updated Safety Analysis Report

PROPOSED CHANGE

Change the word "Final" in Final Safety Analysis Report in TS.6.2.B.4(b) to "Updated".

REASON FOR CHANGE

To indicate the USAR will be used as the document for determining which modifications will require Operations Committee Review. Refer to our letter dated June 18, 1982.

SAFETY EVALUATION

None is required.

5. Specification: TS.6.5.A Plant Operations

PROPOSED CHANGE

Delete the drill requirement statement from TS 6.5.A.

REASON FOR CHANGE

Emergency Plan Drill requirements are specifically detailed in 10 CFR 50 Appendix E and in the Facility Emergency Plans submitted to the NRC for review and determination of their adequacy. The existing requirement is inconsistent with Appendix E requirements and is unnecessary since current requirements are contained in the regulations.

SAFETY EVALUATION

None is required.

6. Specification: TS 6.7 Reporting Requirements

6.1 Failures or Challenges to Safety Valves

PROPOSED CHANGE

A requirement is added to Technical Specification to list in the annual report any safety or relief valve failure or challenges. Refer to Exhibit B, page TS.6.7-2.

REASON FOR CHANGE

To comply with the NRC request to formalize this reporting requirement.

SAFETY EVALUATION

None required since it is an administrative change in reporting requirements.

6.2. Fire Protection Reporting Requirements

PROPOSED CHANGE

Revised Specification 6.7.B to clarify the reporting requirements for fire protection related events. Refer to Exhibit B, page TS.6.7-2.

REASON FOR CHANGE

Fire protection system reporting requirements have been separately specified in the fire protection sections of the Technical Specifications (Sections 3.14 and 4.16).

This clarification clearly states that the established reporting procedures for reportable occurrences does not apply to fire protection systems. The proposed wording is similar to wording previously approved for our Monticello plant.

SAFETY EVALUATION

None required.

7. Specification: TS 3.3.A Engineered Safety Features

PROPOSED CHANGE

Clarify the wording of Specifications TS 3.3.A.1.b and TS 3.3.A.2.e. Refer to Exhibit B, pages TS.3.3-1 and TS.3.3-2.

REASON FOR CHANGE

The change is needed to remove the confusion when interpreting the statements in TS 3.3.A.1.b and TS 3.3.A.2.e. They seem to contradict each other.

SAFETY EVALUATION

Since it is permissible to close an accumulator isolation valve when reactor coolant system pressure is less than 1000 psig, the other conditions related to accumulator operability should not be required until the valve is opened.

8. Specification TS 3.3.D Engineered Safety Features

PROPOSED CHANGE

Replace the word "that" in paragraph a,(2) on page TS.3.3-5A with "the operable diesel driven."

REASON FOR CHANGE

To eliminate any confusion.

SAFETY EVALUATION

This change provides clarification of an existing Technical Specification requirement.

9. Specification: TS 3.4.A Steam Exclusion System

PROPOSED CHANGE

Add words to TS 3.4.A.3 page TS.3.4-2 as noted in Exhibit B.

REASON FOR CHANGE

The proposed change would extend the general philosophy of a limited period of inoperability for one safeguards train to the steam exclusion system. This eliminates problems with disruption of ventilation and excessive damper cycling during tests.

SAFETY EVALUATION

Safeguards trains can be out of service when testing. The same philosophy is extended to the steam exclusion system. The test procedure now cycles each damper six times. One operational check of each damper per month should be adequate. The ability to maintain the operable/redundant damper open during testing or maintenance will permit cooling to be continued to that area.

10. Specification: TS 3.4.A.2 Auxiliary Feedwater System

PROPOSED CHANGE

The condensate supply cross connect valves to the auxiliary feedwater system are to be controlled as noted on page TS.3.4-2 of Exhibit B.

REASON FOR CHANGE

To fulfill commitments to the NRC for NUREG-0737 Items II.E.1.1 and II.E.1.2 to assure there are two level indicators monitoring the condensate storage tanks.

SAFETY EVALUATIONS

By maintaining these valves open, the three condensate storage tanks are available to all auxiliary feedwater pumps. The tanks will have two redundant channels of level indication available when the cross tie valves are open.

11. Specification: Instrumentation System Table TS.4.1-1

Addition to Table TS.4.1-1

PROPOSED CHANGE

Add to Technical Specifications the instrument operating condition limits regarding auxiliary feedwater pump suction and discharge pressure protection as noted in Table TS.4.1-1 Exhibit B.

REASON FOR CHANGE

This change is needed to fulfill NSP commitments to the NRC for satisfying NUREG-0737 Item II.E.1.1 and II.E.1.2 requirements as noted in the Staff's Safety Evaluation Report.

SAFETY EVALUATIONS

This instrumentation is required to protect the auxiliary feedwater pump from damage due to loss of suction or run out. The specified calibration schedule will insure that this instrumentation is available to trip the pump to protect it from damage.