

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE September 15, 1982
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: August, 1982
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.	5831.	70104.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	44.5
20. Unit Availability Factor	0.0	0.0	44.5
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	43.4
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	41.5
23. Unit Forced Outage Rate	100.0	100.0	50.3
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-I
 DATE Sept. 15, 1982
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MONTH August, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August, 1982

DOCKET NO. 50-289
UNIT NAME TMI-I
DATE Sept. 15, 1982
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	8/1/82	F	744	D	1				Regulatory Restraint Order

1

F- Forced
S- Scheduled

2

Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3

Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4

Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NRC/REG-
0161)

5

Exhibit I - Same Source

OPERATING SUMMARY

The unit was shut down the entire report period by order of the NRC. The Reactor Coolant System was partially drained to permit continued inspections of the OTSG's. Core cooling was provided by the decay heat removal system.

MAJOR SAFETY-RELATED MAINTENANCE

Restart modifications continued through the month. Other maintenance included the following:

1. The once-through steam generator (OTSG) program continued with eddy current testing on both the "A" and "B" generators. Additional testing performed this period included cal rod heater testing and crevis humidity tests.
2. The replacement of valve WDL-V-307 was completed, with all NDE and operational tests being performed satisfactorily.
3. A scheduled decay heat (Loop "A") outage commenced with the repacking of various valves, cleaning and resetting of various relief valves, electrical PM's (breaker testing, meggering, oil changes and greasing), instrumentation calibrations (tech spec and PM) and the following major maintenance items:
 - a. DC-C-2A
 1. Performed leak check on tubes.
 2. Cleaned tubes, end covers and water boxes.
 3. Applied PC-7 and coal tar Epoxy on end covers and water boxes.
 4. Installed magnesium anodes on end covers.
 - b. DC-C-1A
 1. Cleaned inlet/outlet flanges and replaced gaskets to repair leaks.
 - c. DH-S-1
 1. Cleaned strainer.
 2. Repaired threaded pipe leaks.

- d. DH-P-1A
 - 1. Repaired threaded pipe leaks.
 - 2. Replaced threaded pipe on pump casing and seal welded.
 - 3. Commenced mechanical seal replacement. Will continue into next month.
- 4. Reactor building ventilation fan (AH-E-1) work continued with the following work accomplished:
 - a. AH-E-1B
 - 1. Balanced in fan unit.
 - 2. Satisfactory testing of motor in fan unit.
 - b. AH-E-1A
 - 1. Sandblasted and painted motor.
 - 2. Manufactured and installed dry seal.
 - 3. Overhauled motor.
 - 4. Shop balanced.
 - 5. Shop tested.
 - 6. Installed motor in fan unit.
- 5. Auxiliary and fuel-handling ventilation fan (AH-E-14) work commenced with the following work accomplished:
 - a. AH-E-14C
 - 1. Sandblasted and painted motor.
 - 2. Overhauled motor.
 - 3. Mounted motor in test stand and bump tested.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).