



**Commonwealth Edison**  
LaSalle County Nuclear Station  
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September 8, 1982

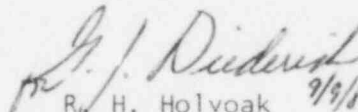
Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station, Unit One, for the period covering August 1 through August 31, 1982.

Very truly yours,

  
R. H. Holyoak 9/9/82  
Superintendent  
LaSalle County Station  
Commonwealth Edison Co.

RHH/JRU/psl

Enclosure

cc: J. G. Keppler - NRC, Region III  
NRC Resident Inspector - LaSalle  
L. O. DelGeorge - CECO  
Gary Wright - Ill. Dept. of Nuclear Safety

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

AUGUST, 1982

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by John Ullrich, telephone number (815)357-6761, extension 481.

## II. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

August 1 Unit was in cold shutdown.

August 2 - 3 Reactor startup commenced at 18:20 hours on August 2, 1982, achieved criticality at 20:00 hours. This criticality was terminated after 26 hours and 55 minutes.

August 4 - 12 The reactor went critical at 7:25 hours on August 4, 1982, for 203 hours and 25 minutes when it was shutdown due to loss of CRD pumps and two accumulator trouble alarms.

August 13 - 17 The reactor went critical on August 13, 1982, at 22:26 hours, for 83 hours and 37 minutes when it was shutdown due to a high differential temperature in the main steam tunnel.

August 18 - 21 The reactor went critical on August 18, 1982, at 15:40 hours, for 62 hours.

August 28 - 31 The reactor went critical at 3:00 on August 28, 1982, for 42 hours and 44 minutes.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED  
MAINTENANCE

- A. The following is a list of Amendments to the facility license and technical specifications.

AMMENDMENT 3. In Mr. A. Schwencer's letter to Mr. Louis O. DelGeorge of July 15, 1982 paragraph 2.C.(31) of the Facility Operating License No. NPF-11 was amended to read as follows:

(31) Bolting of Valves

Prior to January 15, 1983, the licensee shall check the torque on all non-pressure boundary bolts (bolts whose failure will effect the operability of the valve) on each safety-related valve located outside the containment.

AMMENDMENT 4. In Mr. D. Eisenhut's letter to Mr. Louis DelGeorge of Aug. 13, 1982 paragraphs 2.C.(1), 2.C.(11), 2.C.(32), 2.C.(33), 2.D, 2.F and 2.H of the Facility Operating License No. NPF-11 were amended to read as follows:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3323 megawatts thermal).

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED  
MAINTENANCE (CONTINUED)

(11) Environmental Qualifications (Section 3.11, SER, SSER #1, SSER #2)

(a) No later than March 31, 1985, the licensee shall be in compliance with the provisions of NUREG-0588, "Interim Staff Position on Environmental qualifications of Safety-Related Electrical Equipment", for safety-related electrical equipment exposed to a harsh environment.

(b) Complete and auditable records must be available and maintained at a central location which describe the environmental qualification methods used for safety-related electrical equipment in sufficient detail to document the degree of compliance with NUREG-0588. Such records shall be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified to document complete compliance no later than March 31, 1985.

(32) Vacuum Breaker Valves

Prior to November 1, 1982, the licensee shall complete a test and shall submit its evaluation of the results which confirm the capability of the vacuum breaker valves to withstand the opening and closing forces associated with pool swell.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED  
MAINTENANCE (CONTINUED)

(33) Heating-Ventilation and Air Condition Systems

- (a) Prior to exceeding 5% power operation, the licensee must provide formal documentation of information regarding HVAC design fabrication and installation, discussed in meetings with the NRC on August 2 and 4, 1982.
  - (b) Prior to exceeding 50% power operation, the licensee shall submit the results of an independent review acceptable to the NRC staff of the HVAC system, including design changes, fabrication, and installation. The review shall encompass all safety-related HVAC systems and the effect of non-safety related HVAC system failures on safety systems.
- D. Exemptions from certain requirements of Appendices G, H and J and 10 CFR Part 73 are described in the Safety Evaluation Report and Supplement No. 1, No. 2 and No. 3 to the Safety Evaluation Report. In addition, an exemption was requested until the completion of the first refueling from the requirements of 10CFR 70.24 and an exemption from 10 CFR Part 50, Appendix E from performing a full scale exercise within one year before issuance of an operating license, both exemptions are described in Supplement No. 2 of the Safety Evaluation Report. These exemptions are authorized by law and will not endanger life or property or the common defense and security and



III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED  
MAINTENANCE (CONTINUED)

are otherwise in the public interest. Therefore, these exemptions are hereby granted. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, and the rules and regulations of the Commission (except as hereinafter exempted therefrom), and the provisions of the Act.

F. Reporting to the Commission:

(a) The licensee shall report any violations of the requirements contained in Section 2, Items C(1), C(3) through (33), and E of this license within twenty-four (24) hours by telephone and confirmed by telegram, mailgram, or facsimile transmission to the NRC Regional Administrator, Region III, or designee, not later than the first working day following the violation, with a written followup report within fourteen (14) working days.

(b) The licensee shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.

H. This license is effective as of the date of issuance and shall expire April 17, 2022.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY-RELATED  
MAINTENANCE (CONTINUED)

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period.

The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L17265	-----	RHR 1C Service water pump	LTD settings out of adjustment	Affects motor protection	Adjusted LTD settings
L17488	-----	Unit 1 Division 1 Battery Charger AC Breaker	High voltage sensing and alarm board bad	Charger kept tripping	Replaced high voltage sensing and alarm board
L17550	-----	1E31-N008C	Transmitter shifting when subjected to static pressure	Larger than allowed tolerance limits	Replaced transmitter
L17552	-----	1C Outboard MSIV	Limit switch for full closed indication hanging up	Threw limit switch setting out of spec	Cycled valve limits O.K.
L17554	-----	Governor Valve	Incorrect wiring	Light indication is backwards	Wiring conforms to drawing
L17665	-----	RHR A testable check cables	Cables have severely deteriorated	No effect	Replaced sealtite
L17738	-----	ATWS test switches A & B	'+' back-feed to K101 relays	Recirc pump trips while in test position	Lifted T3 lead to prevent breaking from tripping
L17893	-----	OVC05A HVAC compressor	Pressure switch out of adjustment	Compressor kept tripping on low suction pressure	Recalibrated pressure switch
L17894	-----	OVE04CB HVAC compressor	Unit needed to be charged	Compressor kept tripping on low suction pressure	Charged unit with 100 lbs of freon

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L17924	-----	RCIC Barometric Cond. pump	Dirty commutator	Pump does not auto start on Hi level	Cleaned commutator
L17943	82-085/03L-0	DG-2A	Excessive pre-lubing	Oil on cylinders	Cleaned off tops to all 20 pistons
L17999	-----	B IRM	Loose pin on H.V.P.S. connector	Did not trip in "inop" when mode switch removed from operate	Tightened pin on H.V.P.S. connector
L18028	-----	Flow control for panel 1PL75J	Primary containment flow control valve switch out of adjustment	Does not control in AUTO operation	Readjusted switch
L18136	-----	RCIC condensate pump discharge Inboard Isolation Valve	Limit switch not making	Has double indication when closed from panel	Adjusted open limit switch
L18160	-----	'C' IRM Power Supply	Bad preregulator	No effect	Replace preregulator
L18163	82-089/03L-0	RCIC Governor Valve	Dowel pins out of place	Valve sticks at about 50% with a full closed indic- ation. This causes turbine to trip on overspeed	Replaced dowel pins, cleaned control link- age, reset governor level clearance
L18175	82-090/03L-0	RCIC Inboard Equal- izing valve	Limit switch out of adjustment	Valve indicates full open and will not close when cycled	Reset limit switch for proper light indication

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L18267	-----	LPRM 16-57	Incorrect Wiring	LPRM 16-57 detectors B & C readings appear to be swapped	Corrected wiring
L18269	-----	LPRM 24-17	Incorrect wiring	LPRM 24-17 detector C gave a "0" reading while neighboring detectors had positive readings	Corrected wiring
L18271	-----	IRM Channel B	Faulty Hi voltage power supply	Channel reads high compared to other scales	Replaced power supply
L18301	82-091/03L-0	RCIC Steam supply Hi flow switch	Switch out of adjustment	No effect	Adjusted switch
L18316	82-091/03L-0	RCIC Steam supply Hi flow detector	Broken locking device	No effect	Repaired locking device
L18352	82-090/03L-0	Main Steam Mechanical Snubber	Defective snubber	No effect	Replaced snubber
L18362	82-093/03L-0	RCIC Turbine	Sight glass leaks oil from gasket	No effect	Replace gaskets
L18377	-----	1A Battery Charger	Defective pot on sensing board	Output voltage oscillates	Replaced pot
L18380	-----	DW-1 CAM	Loose fitting connector to 1R1T-CM023	Particulate Monitor inoperable	Readjusted fitting connector

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L18385	-----	'B' RHR Injection Line Testable Check Valve	Limits out of adjustment	Valve did not cycle from Control Room	Reset limits
L18387	-----	LPCI 'C' Injection Line Testable Check Valve	Cylinder needed Maint- enance ..	Valve did not cycle com- pletely from Control Room	Replaced software and seals in cylinder
L18524	-----	IRM 'H'	Preregulator voltage low	IRM did not work	Replaced preregulator



#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, August 1 to August 31, 1982. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
82-081/03L-0	8/2/82	RCIC Equipment Area High Temperature Isolation
82-083/01T-0	8/4/82	Unit 2 Division 2 Switchgear Heat Removal System Welds
82-084/03L-0	8/6/82	Steam Leak on 1B21-F080
82-085/03L-0	8/6/82	OG-2A Oil in the Cylinders
82-086/03L-0	8/9/82	Excess of 135° in Drywell Requiring Reactor Shutdown
82-087/03L-0	8/8/82	LIS-NR-02 Functional Test
82-089/03L-0	8/15/82	RCIC Overspeed
82-090/03L-0	8/20/82	Defective Mechanical Snubber
82-091/03L-0	8/18/82	RCIC Steam Line Hi Flow Isolation
82-093/03L-0	8/21/82	Broken Oil Sightglass on RCIC Turbine
82-094/03L-0	8/21/82	1A RR Pump Seals Failure
82-095/03L-0	8/22/82	0B Diesel Fire Pump
82-096/03L-0	8/30/82	RCIC Testable Check Valve
82-097/03L-0	8/16/82	1E51-F355, 1E51-F066, 1E51-F354, 1E51-F065 (Failure to Close)
82-098/03L-0	8/26/82	Mechanical Snubber Not Installed

## V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions