



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

September 3, 1982

Mr. R. C. Haynes  
Regional Administrator  
USNRC  
Region 1  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. Haynes:

LICENSE NO. DPR-75  
DOCKET NO. 50-311  
REPORTABLE OCCURRENCE 82-079/03L

Pursuant to the requirements of Salem Generating Station  
Unit No. 2, Technical Specifications, Section 6.9.1.9.b,  
we are submitting Licensee Event Report for Reportable  
Occurrence 82-079/03L. This report is required within  
thirty (30) days of the occurrence.

Sincerely yours,

H. J. Midura  
General Manager -  
Salem Operations

RH:ks 742

CC: Distribution

8209160498 820903  
PDR ADOCK 05000311  
S PDR

The Energy People

IE22

Report Number: 82-079/03L  
Report Date: 09-03-82  
Occurrence Date: 08-11-82  
Facility: Salem Generating Station, Unit 2  
Public Service Electric & Gas Company  
Hancocks Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Unidentified Leakage in Containment - Greater than 1 GPM.

This report was initiated by Incident Report 82-226.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 - Rx Power 79% - Unit Load 900 MWe.

DESCRIPTION OF OCCURRENCE:

At 1115 hours, August 11, 1982, during normal operation, the Control Room Operator determined by the Containment Sump Pump run times, that he had indication of greater than 1 GPM of unidentified leakage. Action Statement 3.4.7.2b was entered at 1115 hours, and a containment entry was made to identify the source of leakage.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

Investigation revealed that Valve 24GB3 was leaking at the flange .35 GPM, and No. 22 Steam Generator Steam Flow Trap was leaking .25 GPM. Because this leakage was not previously identified on Surveillance Procedure 4.4.7.2, it had raised the unidentified leakage volume to greater than 1 GPM.

ANALYSIS OF OCCURRENCE:

The reactor coolant leakage detection systems are provided to monitor and detect leakage from the Reactor Coolant Pressure Boundary. Industry experience has shown that while a limited amount of leakage is expected from the Reactor Coolant System, the unidentified portion of this leakage can be reduced to a threshold value of less than 1 GPM. This threshold value is sufficiently low to ensure early detection of additional leakage. The 10 GPM identified leakage limitation provides allowance for a limited amount of leakage from known sources whose presence will not interfere with the detection of unidentified leakage by the leakage detection systems. Therefore, leakage from identified and unidentified sources must be maintained within the limits specified by the Technical Specifications, to maintain the capability of detecting possible pressure boundary leakage.

ANALYSIS OF OCCURRENCE: (continued)

Pressure boundary leakage of any magnitude is unacceptable since it may be indicative of an impending gross failure of the pressure boundary.

Action Statement 3.4.7.2b requires:

With any Reactor Coolant System leakage greater than 1 GPM unidentified leakage, or 10 CFM identified leakage, excluding pressure boundary leakage and leakage from Reactor Coolant System Pressure Isolation Valves, reduce the leakage rate to within limits within 4 hours or be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours.

Operation with unidentified leakage of greater than 1 GPM, therefore constituted operation in a degraded mode permitted by a limiting condition for operation, and is reportable in accordance with Technical Specification 6.9.1.9.b.

As noted, investigation revealed that Valve 24GB3 was leaking at the flange, and No. 22 Steam Generator Steam Flow Trap was leaking. The volume of this leakage was identified on Surveillance Procedure 4.4.7.2 and subtracted from the Containment Sump Pump run time leakage indication. The resulting unidentified and identified leakage rates were determined to be within the limits permitted by the Technical Specifications. Therefore, Action Statement 3.4.7.2b was terminated at 1335 hours, August 11, 1982. All leakage was contained and within the limits allowed by the Technical Specification. Therefore, this occurrence involved no risk to the health and safety of the general public.

CORRECTIVE ACTION:

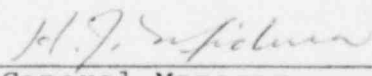
As noted, a containment entry was made to determine the source of leakage. Two sources of leakage were identified and work orders were issued to correct the problems.

FAILURE DATA:

Velan Valve Corp.  
3" Gate Valve  
Type B10-254B-2TS

Velan Valve Corp.  
1" Steam Trap  
Type N-4

Prepared By R. Heller

  
General Manager -  
Salem Operations

SORC Meeting No. 82-80