

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 0 H D B S 1 2 0 0 - 0 0 N P F - 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CON'T  
0 1 REPORT SOURCE L 6 0 5 0 - 0 3 4 6 7 1 0 0 1 7 8 8 1 0 2 7 7 8 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 At 0625 hours on 10/1/78, during performance of surveillance testing, the calculated  
0 3 value for unidentified Reactor Coolant System (RCS) leakage exceeded the 1 GPM limit  
0 4 by 0.08 GPM. This placed the unit in the Action Statement of Technical Specification  
0 5 (T.S.) 3.4.6.2. At 0730 hours on 10/1/78, the increased leakage was identified by a  
0 6 measurement of the Reactor Coolant Pumps standpipe leak-off. This reduced the uniden-  
0 7 tified leakage to 0.54 GPM, and removed the unit from the Action Statement of T.S.  
0 8 3.4.6.2. The small leakages were processed through the normal containment sump.  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

0 9 SYSTEM CODE C 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE Z Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16 (NP-33-78-118)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Increased Reactor Coolant Pump seal leakage is the cause of this occurrence. Investi-  
1 1 gation into the possibility of installing more accurate seal leakage indication is  
1 2 continuing per Facility Change Request 78-357.  
1 3  
1 4  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 5 FACILITY STATUS G 20 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance Test ST 5042.02 32  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 6 ACTIVITY CONTENT Z 33 RELEASED OF RELEASE Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

2 0 PUBLICITY ISSUED N 44 DESCRIPTION NA 45  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

781121035

DVR 78-153 NAME OF PREPARER Scott Fulmer PHONE 419-259-5000, Ext. 276

TOLEDO EDISON COMPANY  
DAVIS-BESSE UNIT ONE NUCLEAR POWER STATION  
SUPPLEMENTAL INFORMATION FOR LER NP-33-78-118

DATE OF OCCURRENCE: October 1, 1978

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Reactor Coolant System (RCS) Unidentified Leakage  
Exceeded the Allowable Limit (1 GPM)

Conditions Prior to Occurrence: The unit was in Mode 3, with Power (MWT) = 0, and Load (MWE) = 0.

Description of Occurrence: At 0625 hours on October 1, 1978, during routine performance of Surveillance Test ST 5042.02, "RCS Water Inventory Balance", the calculated value for unidentified leakage exceeded the 1 GPM limit by 0.08 GPM. This placed the unit in the Action Statement of Technical Specification 3.4.6.2.

At 0730 hours on October 1, 1978, measurement of the Reactor Coolant Pumps standpipe leak-off was found to have increased by .56 GPM. This identified seal leakage reduced the unidentified leakage to  $< 1$  GPM and removed the unit from the Action Statement of Technical Specification 3.4.6.2. | 1

Designation of Apparent Cause of Occurrence: The cause of this occurrence is attributed to changes in the amount of seal leak-off.

Analysis of Occurrence: There was no threat to the health and safety of the public or unit personnel. The final data showed all leakage specifications to be within allowable limits. Additionally, the revised measurement of Reactor Coolant Pump standpipe leakage accounted for essentially all of the observed increase in total leakage. The small leakages were processed through the normal containment sump.

Corrective Action: Prompt action was initiated to investigate and correct the cause of the increase in RCS leakage. Measurement of the seal leak-off by recording the time to fill a 400 ml cylinder showed an overall increase of 0.56 GPM. This increase accounted for essentially all of the observed increase. Investigation into the possibility of installing more accurate seal leakage indication is continuing per Facility Change Request 78-357. | 1

Failure Data: On March 14, 1978, Reactor Coolant System leakage exceeded 1 GPM as a result of an increase in Reactor Coolant Pump seal leak-off (Licensee Event Report NP-33-78-31).