PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

May 13, 1991

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATION STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for April are being forwarded to you with the summary of changes, tests, and experiments for April 1991 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

S. J. Hagan
General Manager -
Hope Creek Operations

RAR:ld
Attachments

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# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/13/91
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH April 1991

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1. | 1032 | 17. | 1056 |
| 2. | 1086 | 18. | 1066 |
| 3. | 1078 | 19. | 1063 |
| 4. | 1070 | 20. | 1041 |
| 5. | 1060 | 21. | 1083 |
| 6. | 1000 | 22. | 1069 |
| 7. | 1008 | 23. | 1051 |
| 8. | 1052 | 24. | 1045 |
| 9. | 1044 | 25. | 1069 |
| 10. | 1057 | 26. | 1067 |
| 11. | 1066 | 27. | 1041 |
| 12. | 1066 | 28. | 1048 |
| 13. | 1046 | 29. | 1053 |
| 14. | 1064 | 30. | 1046 |
| 15. | 1068 | 31. | N/A |
| 16. | 1055 | | |

OPERATING DATA REPORT

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/13/91
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

OPERATING STATUS

1. Reporting Period April 1991 Gross Hours in Report Period 719

2. Currently Authorized Power Level (MWt) 3293
   Max. Depend. Capacity (MWe-Net) 1031
   Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

| | | This Month | Yr To Date | Cumulative |
|---|---|---|---|---|
| 5. | No. of hours reactor was critical | 719.0 | 1578.8 | 31,360.3 |
| 6. | Reactor reserve shutdown hours | 0.0 | 0.0 | 0.0 |
| 7. | Hours generator on line | 719.0 | 1497.7 | 30,790.8 |
| 8. | Unit reserve shutdown hours | 0.0 | 0.0 | 0.0 |
| 9. | Gross thermal energy generated (MWH) | 2,366,890 | 4,663,622 | 97,206,030 |
| 10. | Gross electrical energy generated (MWH) | 789,010 | 1,547,590 | 32,169,263 |
| 11. | Net electrical energy generated (MWH) | 759,565 | 1,470,700 | 30,727,384 |
| 12. | Reactor service factor | 100.0 | 54.8 | 82.0 |
| 13. | Reactor availability factor | 100.0 | 54.8 | 82.0 |
| 14. | Unit service factor | 100.0 | 52.0 | 80.5 |
| 15. | Unit availability factor | 100.0 | 52.0 | 80.5 |
| 16. | Unit capacity factor (using MDC) | 102.5 | 49.5 | 78.0 |
| 17. | Unit capacity factor (Using Design MWe) | 99.0 | 47.9 | 75.3 |
| 18. | Unit forced outage rate | 0.0 | 12.3 | 5.8 |

19. Shutdowns scheduled over next 6 months (type, date, & duration):
    None
20. If shutdown at end of report period, estimated date of start-up:
    N/A

OPERATING DATA REPORT

UNIT SHUTLOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/13/91
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH April 1991

| NO. | DATE | TYPE F=FORCED S=SCHEDULED | DURATION (HOURS) | REASON (1) | METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2) | CORRECTIVE ACTION/COMMENTS |
|---|---|---|---|---|---|---|
| | | | | | | NONE |

Summary

REFUELING INFORMATION

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/13/91
COMPLETED BY S. Hollingsworth
TELEPHONE (609) 339-1051

MONTH April 1991

1. Refueling information has changed from last month:

   Yes No X

2. Scheduled date for next refueling: 9/5/92

3. Scheduled date for restart following refueling: 11/3/92

4. A. Will Technical Specification changes or other license amendments be required?

   Yes No X

   B. Has the reload fuel design been reviewed by the Station Operating Review Committee?

   Yes No X

   If no, when is it scheduled? not currently scheduled

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

   - Amendment 34 to the Hope Creek Tech Specs allows the cycle specific operating limits to be incorporated into the CORE OPERATING LIMITS REPORT; a submittal is therefore not required.

7. Number of Fuel Assemblies:

   A. Incore 764
   B. In Spent Fuel Storage (prior to refueling) 496
   C. In Spent Fuel Storage (after refueling) 760

8. Present licensed spent fuel storage capacity: 4006

   Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: July 22, 2007

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

APRIL 1991

Hope Creek entered the month of April at approximately 100% power and operated for the entire month without experiencing any shutdowns or any reportable power reductions. On April 30th, the unit completed its 59th day of continuous power operation.

# SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE HOPE CREEK GENERATING STATION

APRIL 1991

The following Design Change Packages (DCP's) have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The DCP's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The DCP's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

| DCP | Description of Design Change Package |
|---|---|
| 4EC-3108 | This DCP added two platforms at elevation 88' in the Reactor Building. The platforms are supported off the Drywell wall and are hung from the elevation 99' 5" floor framing steel. The platforms will enhance personnel safety. |
| 4HC-0171 | This DCP added two permanent demineralizer systems, one for each of the Safety Auxiliaries Cooling System loops. The DCP will help to maintain low conductivity water for the Safety Auxiliaries Cooling System Demineralized Water. |
| 4HC-0316 | This DCP installed a Halon storage rack, bottles, piping, pipe supports, penetration seals, and a hose reel. It also replaced nozzles in the Halon Fire Suppression system. This DCP allows the cylinders to be permanently mounted in the Control Area Computer Room and will improve fire-fighting capabilities by reducing the response time. |

The following procedure revisions have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The procedure revisions did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The procedure revisions did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

| Procedure Revision | Description of Procedure Revision |
|---|---|
| HC.OP-IS.EA-0001(Q) Rev. 10 | This procedure revision provides a new attachment to determine if service water pump flow is bypassing through the discharge and strainer of the 'C' Service Water Pump. It also adds instructions to isolate 'A' Loop Service Water Dilution Flow to the Hypochlorination System prior to the test, ensuring that all flow is accounted for during the inservice test. The revision adds steps to isolate the Strainer Keep Fill during the inservice test and changes the flow reference point and the required pressures for the pump. These changes comply with the ASME code. |
| HC.SA-AP.ZZ-0002(Q) Rev. 15 | This procedure revision changes the position titles of personnel in the Operations Department (Nuclear Shift Technical Advisor to Shift Technical Advisor and Radwaste Senior Shift Support Supervisor to Senior Operations Support Supervisor). It also changes the reporting relationship of the Project Manager - Outages and the Shift Support Supervisor. |