

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY
Monticello Nuclear Generating Plant

Docket No. 50-263

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NO. DPR-22

(License Amendment Request Dated March 12, 1975)

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This request contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By L. J. Wachter
L J Wachter
Vice President, Power Production &
System Operation

On this 12th day of March, 1975, before me a notary public in and for said County, personally appeared L J Wachter, Vice President, Power Production & System Operation, and first being duly sworn acknowledged that he is authorized to execute this document in behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interpreted for delay.

John A. Smith
JOHN A. SMITH
Notary Public, Hennepin County, Minnesota
My Commission Expires March 2, 1976

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EXHIBIT A

MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

LICENSE AMENDMENT REQUEST DATED MARCH 12, 1975

PROPOSED CHANGES TO TECHNICAL SPECIFICATIONS
APPENDIX A OF PROVISIONAL OPERATING
LICENSE NO. DPR-22

Pursuant to 10CFR50.59 the holders of Provisional Operating License DPR-22 hereby propose the following changes to Appendix A, Technical Specifications.

The reason for most changes is the adoption of the GETAB/GEXL thermal analysis as a new basis for safety limits. This new, improved correlation replaces the Hensch-Levy heat flux correlation which presently serves as the basis for safety limits. The reason for change stated below as "Adoption of GETAB/GEXL" implies this fact. Additional supporting analysis is presented in the following references:

- i) General Electric BWR Thermal Analysis Basis (GETAB):
Data, Correlation and Design Application; NEDO-10958
- ii) General Electric BWR Generic Reload Application for
8x8 Fuel, NEDO-20360, Revision 1
- iii) Monticello Nuclear Generating Plant, Third Reload
Submittal, December 11, 1974

1. SPECIFICATION 1.0.1 (page 2)

PROPOSED CHANGE

Delete the definition of MCHFR and insert the definition of MCPR as indicated.

REASON FOR CHANGE

Adoption of GETAB/GEXL

Note: Our License Amendment Request Dated August 20, 1974, proposed to change this Specification in response to 10CFR50 Appendix K. The change proposed above supercedes both the existing and the August 20, 1974, proposed versions.

SPECIFICATIONS 2.1 and 2.3 and BASES 2.1 and 2.3 (pages 6-8, 10 and 13-22A)

PROPOSED CHANGES

Revise the pages to conform to the corrected pages included as Exhibit B. Changes are indicated by sidelines. Page 10 (previously Figure 2.1.1) has been deleted due to the change to Specification 2.1.A.

REASON FOR CHANGE

Adoption of GETAB/GEXL

3. BASES 2.1 (page 17)

PROPOSED CHANGE

Delete the last paragraph of this section, found on page 17 of existing Bases.

REASON FOR CHANGE

This section has become obsolete. It served as the Bases for a reporting requirement on the "Summary Status of Fuel" which has been proposed to be eliminated in License Amendment Request Dated December 16, 1974.

4. BASES 3.2/4.2 (page 67)

PROPOSED CHANGE

Change the third sentence on this page to read, "The trip settings of 200°F and 150% of HPCI and 300% of RCIC design flows and valve closure times are such that the core will not be uncovered and fission product release will not exceed 10CFR100 guidelines."

REASON FOR CHANGE

This change corrects an erroneous statement concerning the RCIC high steam flow setpoint. The trip setting for RCIC High Steam Flow listed on page 51 is less than or equal to 45,000 lb/hr. Design flow is 16,500 lb/hr maximum.

5. BASES 3.2/4.2 (page 67)

PROPOSED CHANGES

In addition to the change in item 4 above, make the remaining changes as indicated in the modified page in Exhibit B. Changes are indicated by sidelines.

REASON FOR CHANGES

Adoption of GETAB/GEXL

6. BASES 3.3/4.3 (page 85)

PROPOSED CHANGES

Revise the page to conform to the corrected page included in Exhibit B. Changes are indicated by sidelines.

REASON FOR CHANGE

Adoption of GETAB/GEXL.

Note: Our Change Request Dated March 1, 1974, proposed to change the same subject matter on page 85 as discussed above. Since that request has not been acted on, we request that the attached changes to page 85 supersede the earlier request and that page 85 be considered withdrawn from Change Request Dated March 1, 1974.

7. SPECIFICATIONS 3.11 AND 4.11 (pages 189B thru 189D and 189K) AND BASES 3.11 AND 4.11 (pages 189E thru 189G)

PROPOSED CHANGES

Revise the pages to conform to the corrected pages included in Exhibit B. Changes are indicated by sidelines. Page 189K (Figure 3.11.2) has been added due to the change in Specification 3.11.C.

REASON FOR CHANGES

Adoption of GETAB/GEXL. The Operating MCPR proposed for Specification 3.11.C is based on the analysis of abnormal operational transients. It is more restrictive than that proposed previously which was the initial condition of LOCA calculations. The required Operating MCPR Limit for 8x8 fuel was erroneously reported in Reference iii to be 1.36. The proposed LCO correctly states the limit to be MCPR of 1.41.

The surveillance requirements have been increased to acknowledge the more restrictive limit. The same surveillance requirements have been applied to all fuel limits addressed in sections 3.11 and 4.11. The increase in surveillance frequency requires that the daily surveillance be augmented by a program specified by the plant technical staff. The necessity for such a program and the frequency at which surveillance should be done is a function of the power level, amount and rate of power change, the associated xenon transient, the control rod sequence and pattern, the core exposure and the margin between operating conditions and limiting values. Because of the interrelationship of all these parameters, a reasonable augmented surveillance can only be determined on a case-by-case basis by the plant technical staff.

Note: Specifications 3.11 and 4.11 were formed by our License Amendment Request Dated August 20, 1974. By order of 10 CFR Part 50.46 the more restrictive of the existing Technical Specifications and those proposed on August 20, 1974, are in effect. The present changes, upon issuance, will supersede the respective portions of the August 20, 1974, submittal. However, since the present changes are not the result of further ECCS evaluations, they are not covered by 10 CFR Part 50.46 and therefore will not become effective until issued by the Nuclear Regulatory Commission.

EXHIBIT B

LICENSE AMENDMENT REQUEST DATED MARCH 12, 1975

Exhibit B, attached, consists of newly prepared pages for the Appendix A Technical Specifications as listed below. These pages incorporate the proposed changes.

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22a

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189B

189C

189D

189E

189F

189G

189K