

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

Monticello Nuclear Generating Plant

Docket No. 50-263

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NO. DPR-22

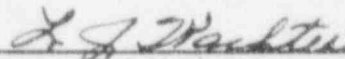
(License Amendment Request Dated March 24, 1975)

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This request contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By



L. J. Wachter

Vice President, Power Production &
System Operation

On this 24th day of March, 1975, before me a notary public in and for said County, personally appeared L. J. Wachter, Vice President, Power Production & System Operation, and first being duly sworn acknowledged that he is authorized to execute this document in behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.



JOHN J. SMITH

Notary Public, Hennepin County, Minnesota
My Commission Expires March 3, 1976

EXHIBIT A

MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

AMENDMENT REQUEST DATED MARCH 24, 1975

PROPOSED CHANGES TO THE TECHNICAL SPECIFICATIONS
APPENDIX A OF OPERATING LICENSE NO. DPR-22

Pursuant to 10CFR50.59, the holders of the above mentioned license hereby propose the following changes to Appendix A Technical Specifications:

1. Specification 3.7.A.1

Proposed Changes

Change Specification 3.7.A.1 to read:

1. Whenever primary containment is required, the volume and temperature of the water in the suppression chamber shall be maintained within the following limits:
 - (a) Maximum water volume - 77,970 cubic feet.
 - (b) Minimum water volume - 68,000 cubic feet.
 - (c) Maximum water temperature
 - i. During power operation, except as specified in item 3.7.A.1.(c).ii below - 90°F.
 - ii. During testing which adds heat to the suppression pool - 100°F.
 - iii. Following a scram from a condition where the suppression pool temperature is 90°F or less, the reactor pressure vessel shall be depressurized at normal cooldown rates to less than 200 psig if the suppression pool temperature exceeds 120°F.
 - iv. During power operation if the temperature reaches 110°F the reactor shall be scrammed immediately and depressurized at normal cooldown rates to less than 200 psig.

Exhibit A

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- (d) In order to continue power operation after exceeding the limit of 1(c)i, the suppression pool temperature must be reduced to 90°F within 24 hours. If the temperature cannot be reduced within this time limit, normal reactor shutdown procedures shall be initiated.

Reason for Change

To minimize the probability of steam quenching vibration in the suppression pool by precluding the development of elevated temperatures in the pool.

The temperature limits specified in this proposed change were based on conservative engineering judgments by the General Electric Company. An analysis is planned that will establish a definitive set of temperature limits based on the Monticello plant configuration. When this analysis is completed the new limits will be reflected in a further technical specification change request.

2. Specification 4.7.A.1

Proposed Changes

Change Specification 4.7.A.1 to read:

1. Checks and inspections of the suppression chamber structures, water level and temperature shall be conducted as follows:
 - (a) Water level and temperature shall be checked once per day.
 - (b) The interior painted surfaces above the water level shall be inspected at each refueling outage.
 - (c) Whenever there is indication that there was relief valve operation with the temperature of the suppression pool exceeding 160°F and the reactor vessel pressure greater than 200 psig, an external visual examination of the pressure suppression chamber shall be conducted before resuming power operation.

Reason for Change

Specification 4.7.A.1 (c) has been added to provide assurance that no significant damage resulted from the occurrence described in the specification.

Exhibit A

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3. Bases 3.7.A

Proposed Changes

- a. Change the next to last line of the third full paragraph on page 157 to read:

"...restricted by Specification 3.7.A.1(c) by limiting the suppression pool initial temperature and the..."

- b. Add the following paragraph after the third paragraph on page 157:

Experimental data indicates that excessive steam condensing loads can be avoided if the peak temperature of the pressure suppression pool is maintained below 160°F during any period of relief valve operation with sonic conditions at the discharge exit. Specifications have been placed on the envelope of reactor operating conditions so that the reactor can be depressurized in a timely manner to avoid the regime of potentially high pressure suppression chamber loadings:

Reason for Changes

To provide bases for the proposed changes to Specification 3.7.A.1.

4. Bases 4.7.A

Proposed Change

Add the following paragraph to page 161:

The requirement for an external visual examination following any event where potentially high loadings could occur provides assurance that no significant damage was encountered. Particular attention should be focused on structural discontinuities in the vicinity of the relief valve discharge since these are expected to be points of highest stress.

Reason for Change

To provide bases for the proposed changes to Specification 4.7.A.1.