

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY
MONTICELLO NUCLEAR GENERATING PLANT

Docket No. 50- 263

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NO. DPR-22

(License Amendment Request Dated September 19, 1977)

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This request contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By

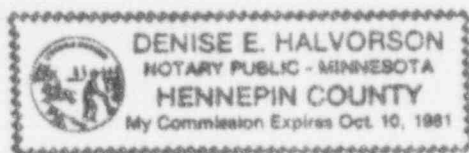
L. J. Wachter

L J Wachter

Vice President, Power Production &
System Operation

On this 19th day of September, 1977, before me a notary public in and for said County, personally appeared L J Wachter, Vice President, Power Production & System Operation, and first being duly sworn acknowledged that he is authorized to execute this document in behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.

Denise E. Halvorson



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EXHIBIT A

MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

License Amendment Request Dated September 19, 1977

Proposed Changes to the Technical Specifications, Appendix A
of Provisional Operating License DPR-22

Pursuant to 10 CFR 50, Section 50.59, the holders of Provisional Operating License DPR-22 hereby propose the following change to Appendix A, Technical Specifications:

Table 4.6.1, "In-Service Inspection Requirements for Monticello"

PROPOSED CHANGE

- a. Change the examination method for category E(2) on page 125 from volumetric to pressure tests.
- b. Add a new note at the end of Table 4.6.1 as follows:
 1. The specified inspection interval and extent of examination are nominal values only. Inservice inspection may be performed during normal plant outages such as refueling shutdowns or maintenance shutdowns occurring during the inspection interval. Except where it is stated that examinations may be deferred to the end of the inspection interval, at least 25% of the required examination shall have been completed by the expiration of one-third of the inspection interval (with credit for no more than 33-1/3% if additional examinations are completed) and at least 50% shall have been completed by the expiration of two-thirds of the inspection interval (with credit for no more than 66-2/3%). The remaining required examinations shall be completed by the end of the inspection interval. Inspection intervals may be extended by as much as one year to permit inspections to be concurrent with plant outages.

REASON FOR CHANGE

These changes are requested on an interim basis until our License Amendment Request dated August 30, 1977 has been reviewed and approved by the Commission. Our August 30, 1977 submittal requested the removal of Table 4.6.1 and required inservice inspection and testing to conform to the requirements of 10 CFR 50, Section 50.55a(g).

Change (a) substitutes examination during system pressure tests for a volumetric examination. Based on Section XI of the ASME Code (1974 Edition with Addenda through Summer 1975), control rod housing pressure boundary welds are Table IWB-2500 Examination Category B-P. They are exempted from volumetric and surface examination by Article IWB-1220(b)(1). Examination during system pressure tests is specified for these components. The requirement for

a volumetric inspection was contained in a preliminary version of the ASME inservice inspection requirements when Table 4.6.1 was drafted. Section XI of the ASME Code had not yet been issued. The 1970 and subsequent editions of the Code have not specified volumetric examination for these components.

Volumetric inspections have been conducted over the last six years at great expense. They have been time consuming, difficult to coordinate with other outage activities, and have resulted in significant occupational radiation exposures. Thermal sleeves removed to perform the volumetric inspection must frequently be renewed. Thermal sleeves that that would be needed for anticipated replacement if volumetric inspection was performed this outage have not been available. A significant delay in return to service could result if we must perform the volumetric inspection and replacement thermal sleeves cannot readily be found.

We have been informed by our reactor vendor that Monticello is the only BWR known to have a Technical Specification requirement for volumetric inspection of these components.

Change (b) is being submitted following discussions with our licensing project manager. Table 4.6.1 has always been liberally interpreted and specified inspection intervals have been adjusted to conform to refueling outages. The majority of the specified inspections can only be performed during refueling. The specified intervals in the "Inspection Interval" column and the "Extent of Examination" column cannot be strictly adhered to.

The requested change adds a new page to the table with a note that indicates intervals may be adjusted to be concurrent with plant outages. This note is consistent with the requirements of 10 CFR 50, Section 50.55a(g). The note simply states the degree of flexibility now permitted by Section XI of the ASME Code.

Strict conformance to Table 4.6.1 would necessitate frequent shutdowns for inservice inspection to comply with an arbitrary inspection schedule.

SAFETY EVALUATION

The requested changes are consistent with the requirements of 10 CFR 50, Section 50.55a(g) and Section XI of the ASME Code (1974 Edition with Addenda through Summer 1975). Change (a) replaces one inspection method with a completely satisfactory alternate method. Change (b) revises the table specifying inservice inspection intervals to include the ASME Code allowable flexibility in scheduling inspections. There are no nuclear safety implications.

EXHIBIT B

License Amendment Request Dated September 19, 1977

This exhibit consists of the following pages revised to incorporate the proposed Technical Specification changes:

Page 125
129A (new page)