



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Operations

December 14, 1990

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATION STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for November are being forwarded to you with the summary of changes, tests, and experiments for November 1990 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. J. Hagan
General Manager -
Hope Creek Operations

RAR:ld
Attachments

C Distribution

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PDR ADOCK 05000354
R PDR

The Energy People

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
UNIT Hope Creek
DATE 12/14/90
COMPLETED BY M. Zapolski
TELEPHONE (609) 339-3738

MONTH November 1990

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1.	<u>1124</u>
2.	<u>1025</u>
3.	<u>1041</u>
4.	<u>9</u>
5.	<u>0</u>
6.	<u>0</u>
7.	<u>0</u>
8.	<u>0</u>
9.	<u>0</u>
10.	<u>0</u>
11.	<u>0</u>
12.	<u>0</u>
13.	<u>0</u>
14.	<u>0</u>
15.	<u>170</u>
16.	<u>950</u>

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

17.	<u>152</u>
18.	<u>0</u>
19.	<u>96</u>
20.	<u>911</u>
21.	<u>1055</u>
22.	<u>1062</u>
23.	<u>1051</u>
24.	<u>1033</u>
25.	<u>1025</u>
26.	<u>995</u>
27.	<u>1053</u>
28.	<u>1039</u>
29.	<u>1055</u>
30.	<u>1006</u>
31.	

OPERATING DATA REPORT

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 12/14/90
 COMPLETED BY M. Zapolski
 TELEPHONE (609) 339-3738

OPERATING STATUS

1. Reporting Period November 1990 Gross Hours in Report Period 720

2. Currently Authorized Power Level (MWt) 3293
 Max. Depend. Capacity (MWe-Net) 1031
 Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

	<u>This Month</u>	<u>Yr To Date</u>	<u>Cumulative</u>
5. No. of hours reactor was critical	<u>437.2</u>	<u>7418.0</u>	<u>29,179.5</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>396.6</u>	<u>7340.9</u>	<u>28,692.1</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>1,201,978</u>	<u>23,672,603</u>	<u>90,328,979</u>
10. Gross electrical energy generated (MWH)	<u>399,000</u>	<u>7,816,330</u>	<u>29,972,593</u>
11. Net electrical energy generated (MWH)	<u>375,517</u>	<u>7,480,537</u>	<u>28,637,086</u>
12. Reactor service factor	<u>60.7</u>	<u>92.5</u>	<u>84.3</u>
13. Reactor availability factor	<u>60.7</u>	<u>92.5</u>	<u>84.3</u>
14. Unit service factor	<u>55.1</u>	<u>91.6</u>	<u>82.9</u>
15. Unit availability factor	<u>55.1</u>	<u>91.6</u>	<u>82.9</u>
16. Unit capacity factor (using MDC)	<u>50.6</u>	<u>90.5</u>	<u>80.3</u>
17. Unit capacity factor (Using Design MWe)	<u>48.9</u>	<u>87.5</u>	<u>77.6</u>
18. Unit forced outage rate	<u>36.4</u>	<u>5.6</u>	<u>5.6</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration): Refueling, 12/26/90, 52 days			
20. If shutdown at end of report period, estimated date of start-up: N/A			

OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354
UNIT Hope Creek
DATE 12/14/90
COMPLETED BY M. Zapolski
TELEPHONE (609) 339-3738

MONTH November 1990

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
11	11/4	F	270.3	A	3	Inboard MSIV Instrument Gas Leak LER 354/90-024
12	11/17	F	53	A	3	"A" Moisture Separator Drain Tank High Level LER 354/90-028

Summary

REFUELING INFORMATION

DOCKET NO. 50-354
UNIT Hope Creek
DATE 12/14/90
COMPLETED BY S. Hollingsworth
TELEPHONE (609) 339-1051

MONTH November 1990

1. Refueling information has changed from last month:

Yes ☒ No

2. Scheduled date for next refueling: 12/26/90

3. Scheduled date for restart following refueling: 02/13/91

4. A. Will Technical Specification changes or other license amendments be required?

Yes No ☒

B. Has the reload fuel design been reviewed by the Station Operating Review Committee?

Yes No ☒

If no, when is it scheduled? not currently scheduled

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

- Amendment 34 to the Hope Creek Tech Specs allows the cycle specific operating limits to be incorporated into the CORE OPERATING LIMITS REPORT; a submittal is therefore not required.

7. Number of Fuel Assemblies:

A. Incore	<u>764</u>
B. In Spent Fuel Storage (prior to refueling)	<u>496</u>
C. In Spent Fuel Storage (after refueling)	<u>760</u>

8. Present licensed spent fuel storage capacity: 4006

Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: July 22, 2007

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

NOVEMBER 1990

Hope Creek entered the month of November at approximately 100% power. On November 4th, the unit automatically shutdown after completing 221 days of continuous power operation. The scram occurred due to Average Power Range Monitor Fixed Neutron Flux Upscale, which was initiated by the closure of the "B" Inboard Main Steam Isolation Valve. The Main Steam Isolation Valve closure resulted from a failed fitting on a Primary Containment Instrument Gas Supply Line to the valve. The unit was returned to service on November 15th. The unit automatically shutdown on November 17th, due to a high level in the "A" Moisture Separator Drain Tank. The unit was returned to service on November 19th, and on November 30th, completed its 11th day of continuous power operation.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

NOVEMBER 1990

The following Design Change Packages (DCP's) have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The DCP's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The DCP's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

LCPDescription of Design Change Package

- 4EC-3144 This DCP installed a reinforced concrete retaining basin for the Low Volume Oily Waste System's Oily Water Separator and Oil Sludge Tank. The installation of the basin will prevent the discharge of oily water to the environment.
- 4EC-3186 This DCP installed additional temporary instrumentation to monitor vibration levels on the Reactor Recirculation System Pumps Suction Elbows and the inner and outer radius instrument lines attached to the pump suction elbows. The vibration monitoring instrumentation will be used for subsequent testing to measure and record piping strains, accelerations, and displacements.
- 4EC-3199 This DCP replaced schedule 40 pipe nipples with schedule 80 pipe nipples. The nipples are connected to the taps in the aluminum solenoid manifold actuator block of the Inboard Main Steam Isolation Valves. The DCP also added unions and replaced flanges with other fittings. This is an upgrade that increases reliability.
- 4hC-0245/01 This DCP added new flow indicators and replaced existing pressure indicators. This will eliminate the need to use portable Measurement and Test Equipment during the performance of the Diesel Fuel Oil Transfer Pump Surveillance Test.
- 4HM-0654 This DCP modified level switches used to sense asphalt level inside the Extruder Evaporator Steam Domes. The modification will eliminate the high level alarm during normal operations.

The following Temporary Modification Requests (TMR's) have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The TMR's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The TMR's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

TMRDescription of Temporary Modification Request

- 90-054 This TMR removed the overload heaters from the breakers for the Reactor Water Cleanup Discharge to Condenser Valve and the Reactor Water Cleanup Discharge to Equipment Drain Valve. Removing the overload heaters from the breakers will prevent the valves from inadvertently opening during an Appendix R fire.
- 90-055 This TMR removed the overload heaters from the breaker for the Residual Heat Removal Outboard Shutdown Cooling Isolation Valve. Removing the overload heaters from the breaker will prevent the valve from inadvertently opening during an Appendix R fire.
- 90-056 This TMR jumpered the High/High Level Trip from the #2 "A", "B", and "C" Feedwater Heaters. Spurious High Level signals have been observed at low power levels due to inleakage in the reference legs. This TMR was installed only until the level signals stabilized.
- 90-057 This TMR jumpered the High/High Level Trip from the #2 "A", "B", and "C" Feedwater Heaters. Spurious High Level signals have been observed at low power levels due to inleakage in the reference legs. This TMR was installed only until the level signals stabilized.
- 90-072 This TMR removed the overload heaters from the breakers for the Reactor Water Cleanup Discharge to Condenser Valve and the Reactor Water Cleanup Discharge to Equipment Drain Valve. Removing the overload heaters from the breakers will prevent the valves from inadvertently opening during an Appendix R fire.

The following procedure revisions have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The procedure revisions did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The procedure revisions did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Procedure
Revision

Description of Procedure Revision

NC.NA-AP.ZZ-0001(Q)
Rev. 2

This procedure revision moves material to NC.NA-AP.ZZ-0032(Q) and makes several other changes that are administrative in nature. This revision also added a new topic, the Nuclear Department Procedure System, to Section 13.5 of the UFSAR.

NC.NA-AP.ZZ-0003(Q)
Rev. 0

This procedure describes the document control program for the PSE&G Nuclear Department. It changes the overall responsibility for the document control program from the Station General Manager to the General Manager - Information Systems and External Affairs.

NC.NA-AP.ZZ-0032(Q)
Rev. 0

This procedure describes the new process for preparing, reviewing, and approving procedures. This procedure requires a change to Section 13.5 of the UFSAR because it refers to Nuclear Administrative Procedures rather than to Station Administrative Procedures.