NRC FORM 366 (12-81) 10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB 3150-0011

LICENSEE EVENT REPORT

CONTROL BLOCK: [| | | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: A L B R F 3 (2)	LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3)	LICENSE TYPE: 4 1 1 1 (4)	CAT (5)	

CON'T 0 1	REPORT SOURCE: L (6)	DOCKET NUMBER: 0 5 0 0 0 2 9 6 (7)	EVENT DATE: 0 8 0 4 8 3 (8)	REPORT DATE: 0 9 0 2 8 3 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal reactor operation, both off-gas post-treatment radiation
0 3 monitors (3-RM-90-265/266) became inoperable due to sample pump power
0 4 failure (T.S. 3.2.D.1.b and c). Monitors were out-of-service for one
0 5 hour. There was no effect on public health and safety. The stack
0 6 radiation monitor was operable and no release limits were exceeded. A
0 7 review of SI 4.8.B.1.a.1, Airborne Effluent Release Rate, indicated
0 8 no significant increase in building effluent activity levels during this event.

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	I E (11)	E (12)	X (13)	P U M P X X (14)	G (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 4 8	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER (26)
X (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	L (25)	G 0 8 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Sample pump was inoperable due to trip of Motor Control Center A
1 1 normal feeder breaker (GE trip unit TJK636T400) which supplies power to
1 2 the pump. No reason for the breaker trip could be found. Breaker
1 3 was reset. Currents were verified to be below trip value. This was
1 4 a random failure and no recurrence control is required.

1 5	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	E (28)	0 9 6 (29)	NA	A (31)	Control Room Alarm

1 6	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	NA	NA

1 7	PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	NA

1 8	PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
	0 0 0 (40)	NA

1 9	LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
	Z (42)	NA

8309090555 830902
PDR ADOCK 05000296
S PDR

2 0	PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
	N (44)		

NAME OF PREPARER Stan W. Solley PHONE (205) 729-0891

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83048 Technical Specification Involved 3.2.D

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 9/3/83

Event Narrative:

Unit 1 was in a refueling outage, Unit 2 was operating normally at 95 percent power, and Unit 3 was operating at 96 percent power. Only Unit 3 was affected by this event. An off-gas sample flow abnormal alarm was observed. The alarm was due to power loss to the off-gas motor control center (MCC) board A. Since this provides power to the off-gas sample pump this made both radiation monitors (3-RM-90-265/266) inoperable (T. S. 3.2.D.1.b and c).

The loss of power was due to a trip on the normal feeder breaker (GE type TJK636F000 with trip unit TJK636T400). The alternate feeder breaker was closed, then the board was transferred back to the normal feeder breaker. This restored power to the board and all loads were returned to service. The load on the board was later measured to be approximately 100 amps. No reason for the trip of the 400 amp trip unit was found. The off-gas post treatment monitors were inoperable for 1 hour.

There was no effect on public health or safety. During the time both off-gas post treatment monitors were inoperable, the off-gas pre-treatment and stack radiation monitors were operable. No significant increase in activity levels was detected by these monitors during this time. The load on the board was later measured to be approximately 100 amps.

* Previous Similar Events:

BFRO-50-259/78032
296/83027

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

September 2, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE REPORT BFRO-50-296/83048

The enclosed report provides details concerning off-gas post-treatment radiation monitors which became inoperable because of sample pump power failure. This report is submitted in accordance with Browns Ferry unit 3 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Herbert Abercrombie

H. J. Green
Director of Nuclear Power

Enclosure
cc (Enclosure):
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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