PP&L Pennsylvania Power & Light Company
Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

Bruce D. Kenyon
Vice President-Nuclear Operations
215/770-7502

AUG 15 1983

Director, Data Automation &
Management Information Division
Attention: Mr. M. R. Beebe
Management Information Branch
Office of Resource Management
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION
MONTHLY OPERATING REPORT - JULY 1983
ER 100450 FILE 841
PLA-1787

Docket No. 50-387

Dear Mr. Beebe:

The July 1983 monthly operating report for Susquehanna SES Unit 1 is attached. This report includes information on refueling.

Very truly yours,

B. D. Kenyon
Vice President-Nuclear Operations

Attachment

cc: Dr. Thomas E. Murley
Regional Administrator-Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

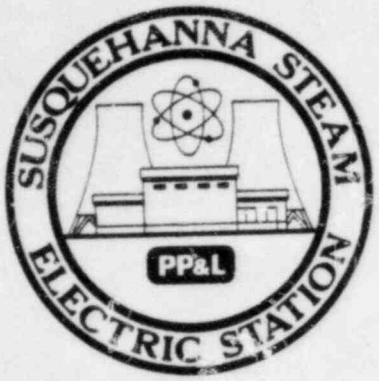
Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555
Attn: Document Control Desk (12 copies)

Mr. G. Rhoads - NRC
Mr. R. Perch - NRC

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

Mr. Thomas E. Pollog
Department of Environmental Resources
Bureau of Radiation Protection
P.O. Box 2063
Harrisburg, PA 17120

IE24
1/1



AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-387
UNIT One
DATE 08-08-83
COMPLETED BY L.A. Kuczynski
TELEPHONE (717)542-2181

MONTH July, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	561
2	25	18	574
3	630	19	556
4	847	20	588
5	544	21	583
6	0	22	549
7	297	23	544
8	563	24	561
9	622	25	756
10	652	26	743
11	428	27	865
12	155	28	874
13	463	29	861
14	531	30	827
15	578	31	1022
16	491		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-387
DATE 08-08-83
COMPLETED BY L.A. Kuczynski
TELEPHONE (717)542-2181

OPERATING STATUS

Unit 1

1. Unit Name: Susquehanna Steam Electric Station
2. Reporting Period: July, 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1072
7. Maximum Dependable Capacity (Net MWe): 1036

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons.
   None
9. Power Level To Which Restricted, If Any (Net MWe): Approx. 85% from 7/25 to 7/29
   Approx. 65% from 7/8 to 7/25
10. Reasons For Restrictions, If Any: Voluntary power level restrictions in effect from July 8 through 29, 1983, due to problems with spiking to three times normal on the main steamline radiation monitors.

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	1,296	1,296
12. Number Of Hours Reactor Was Critical	662	1,013.6	1,013.6
13. Reactor Reserve Shutdown Hours	0	156.7	156.7
14. Hours Generator On-Line	641	975.3	975.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,402,028	2,452,028	2,452,028
17. Gross Electrical Energy Generated (MWH)	434,450	776,210	776,210
18. Net Electrical Energy Generated (MWH)	415,082	744,992	744,992
19. Unit Service Factor	86.2	75.2	75.2
20. Unit Availability Factor	86.2	75.2	75.2
21. Unit Capacity Factor (Using MDC Net)	53.8	55.5	55.5
22. Unit Capacity Factor (Using DER Net)	52.4	54	54
23. Unit Forced Outage Rate	13.8	24.7	24.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    Unit 1/Unit 2 Tie-in Outage to commence November 15, 1983. Scheduled to last 29 days.
25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

SUSQUEHANNA STEAM ELECTRIC STATION
PP&L

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1983

DOCKET NO. 50-387
UNIT NAME One
DATE 08-08-83
COMPLETED BY L.A. Kuczynski
TELEPHONE (717)542-2181

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	062483	F	42.5	A	3	83-092/03L	EA	TRANSF	A startup transformer failed in service. This caused a reactor scram and main turbine trip on RPV high level. Present replacement transformer will be replaced with transformer having automatic load tap changing.
14	070583	F	39.2	H	3	NA	HG	DEMINX	The scram was due to chemistry transients associated with placing condensate demins. in service causing spikes in main steam line radiation.
15	071183	F	21.3	H	3	NA	IA	CKTBRK	The scram was caused by an incorrect fuse type installed in the power supply to one of the scram valve pilot solenoid groups. The fuse was blown at the same time the I&C group was conducting surveillance testing which, by procedure, caused a half scram. This combination of events resulted in (cont.)

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

JULY, 1983

Cause & Corrective Action to Prevent Recurrence: (Continued)

the insertion of one-quarter of all the control rods (48). The resulting power decrease caused a rapid RPV level decrease which yielded a full scram. A new procedure was issued that is expected to prevent recurrence.

SUSQUEHANNA STEAM ELECTRIC STATION

Docket Number 50-387 Date 08-08-83
Completed by L.A. Kuczynski Telephone (717)542-2181

Challenges to Main Steam Safety Relief Valves

SRV's 'A' and 'B' were manually actuated for pressure control at 5 min. and 21 min, respectively, after the July 5 scram. SRV 'A' was opened for approximately 3.3 min; RPV pressure decreased from 1030 psig to 860 psig. SRV 'B' was opened for approximately 4.1 min.; RPV pressure decreased from 1050 psig to 860 psig. Both valves reseated properly.

Changes to the Offsite Dose Calculation Manual

None.

Major Changes to Radioactive Water Treatment Systems

None.

DATE: 5/18/83

REFUELING INFORMATION

1. Name of facility. Susquehanna SES - Unit 1

2. Scheduled date for next refueling shutdown. 9/1/84

3. Scheduled date for restart following refueling. 12/15/84

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
If answer is yes, what, in general, will these be?
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   Yes; first insertion of Exxon Nuclear Co. Fuel bundles, will require NRC review, may change tech specs for MCPR; must change T.S. for MAPLHGR (new ENC fuel bundle)

5. Scheduled date(s) for submitting proposed licensing action and supporting information. 9/1/84

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   New fuel supplier (same basic conceptual design - 8x8C, 2 water rod).

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 764 b) 0

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   present 2840 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   DATE: 1997*

   * based on 2840 - 1/2 X 764 = 2458 available capacity