

**LICENSEE EVENT REPORT**

**CONTROL BLOCK:**

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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7	8	14						15	25										26	30					57	58			
LICENSEE CODE		LICENSE NUMBER										LICENSE TYPE					CAT												

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REPORT SOURCE 1 6 0 5 0 0 0 3 1 1 7 0 7 2 6 8 3 8 0 7 2 9 8 3 9  
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | On July 26, 1983, during routine startup operations, it was discovered that No. 23 ECCS

03 Accumulator level was slightly higher than required by the Technical Specifications.

04 | Review of operating logs revealed that the condition had existed for greater than the

05, interval required by the appropriate action requirement. The level was immediately

06 drained to within specification. The event possibly constituted operation less

07 : conservative than the least conservative aspect of a limiting condition for operation

CS per Technical Specification 6.9.1.8b.

SYSTEM CODE S F 11		CAUSE CODE A 12		CAUSE SUBCODE A 13		COMPONENT CODE A C C U M U 14				COMP SUBCODE Z 15		VALVE SUBCODE Z 16					
EVENT YEAR 8 3 21 22		SEQUENTIAL REPORT NO. 0 3 6 24 26		OCCURRENCE CODE 0 1 28 29		REPORT TYPE T 30		REVISION NO. 0 32									
ACTION TAKEN E 18		FUTURE ACTION H 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB N 24		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER D 1 0 0 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	The increase in accumulator level was associated with startup operation and plant
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11 | heatup. The out of specification level condition was inadvertently overlooked by the

[ ] operator involved; the individual was counseled concerning the appropriate Technical

13 | Specifications and accepted operating practices.

1	4
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FACILITY STATUS		% POWER			OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1	5	C	(28)	0 0 0	(29)	NA	D (31) Notification by NRC Inspector

ACTIVITY		CONTENT		RELEASED		OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	6	2	(33)	2	(34)	NA				NA	

PERSONNEL EXPOSURES		44		45		80	
NUMBER	TYPE	DESCRIPTION (39)					
1	2	37	38				

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	1	1	1
2	2	2	2
3	3	3	3
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LOSS OF OR DAMAGE TO FACILITY		(43)
TYPE	DESCRIPTION	

7 8 9 10  
PUBLICITY  
ISSUED DESCRIPTION (45)  
8308150189 830729  
PDR ADQCK 05000311  
S PDR  
NRC USE ONLY

NRC USE ONLY

NAME OF PREPARER \_\_\_\_\_

R. Frahm

PHONE: (609) 935-6000 Ext. 4309

Report Number: 83-036/01T  
Report Date: 07-29-83  
Occurrence Date: 07-26-83  
Facility: Salem Generating Station Unit 2  
Public Service Electric & Gas Company  
Hancock's Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Emergency Core Cooling Systems - No. 23 Accumulator - Inoperable.

This report was initiated by Incident Report 83-124.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 2 - Rx Power 0 % - Unit Load 0 MWe.

DESCRIPTION OF OCCURRENCE:

At 1210 hours, July 26, 1983, during routine startup operations, a review of the operating logs by an NRC Resident Inspector revealed that the level of No. 23 Emergency Core Cooling System (ECCS) Accumulator at 0800 hours that day was greater than the maximum allowed by the Technical Specifications (72.0% as opposed to the required 70.5%). Further investigation revealed that the level was still out of specification. The accumulator was declared inoperable and Technical Specification Action Statement 3.5.1a was entered retractive to 0800 hours.

The level of the accumulator was immediately decreased by draining and at 1213 hours the contained volume was returned to within Technical Specification requirements. The three remaining accumulators were operable throughout the occurrence; the integrity of the Reactor Coolant System (RCS) pressure boundary was maintained throughout the period and no pressure transients occurred.

APPARENT CAUSE OF OCCURRENCE:

A slow increase in accumulator levels particularly during startup operation is not unusual. Investigation of the incident revealed that the high level condition had escaped notice at the time the level had been recorded on the logs. The item had not been circled with a red pen as required. The occurrence was assumed to involve an isolated oversight on the part of the watchstander involved.

ANALYSIS OF OCCURRENCE:

The operability of each ECCS accumulator ensures that a sufficient volume of borated water will be immediately forced into the reactor core through each of the RCS cold legs in the event the system pressure falls below the pressure of the accumulators. This initial surge of water into the core provides the initial cooling mechanism during large RCS pipe ruptures.

ANALYSIS OF OCCURRENCE: (cont'd)

As noted the RCS integrity was maintained throughout the occurrence, and no pressure transients occurred during the period of concern. The event therefore involved no impact on the health or safety of the public. It must be noted that the small increase in level involved would only slightly decrease the effectiveness of the accumulator. It is also possible that the corresponding increase in pressure resulting from the level increase would maintain accumulator operability. Final evaluation of the incident is presently underway.

If the accumulator was, in fact, inoperable for greater than 4 hours, compliance with Limiting Condition for Operation 3.5.1 would not have been maintained. Due to potential operation of the unit in a manner less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications, the event is reportable in accordance with Technical Specification 6.9.1.8b.

## Action Statement 3.5.1a requires:

With one accumulator inoperable, except as a result of a closed isolation valve, restore the inoperable accumulator to operable status within one hour or be in hot shutdown within the next 12 hours.

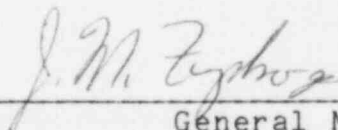
CORRECTIVE ACTION:

As noted, the No. 23 Accumulator level was restored to within specification at 1213 hours, July 26, 1983, and Action Statement 3.5.1a was terminated. The individual involved was counseled concerning the appropriate Technical Specifications and related accepted operating practices. A Supplemental Report will be submitted upon completion of evaluation of the incident.

FAILURE DATA:

Not Applicable

Prepared By R. Frahm

  
\_\_\_\_\_  
General Manager -  
Salem Operations

SORC Meeting No. 83-102



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 4, 1983

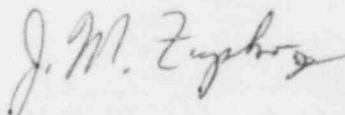
Dr. Thomas E. Murley  
Regional Administrator  
USNRC  
Region 1  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Dr. Murley:

LICENSE NO. DPR-75  
DOCKET NO. 50-311  
REPORTABLE OCCURRENCE 83-036/01T

Pursuant to the requirements of Salem Generating Station  
Unit No. 2, Technical Specifications, Section 6.9.1.8b,  
we are submitting Licensee Event Report for Reportable  
Occurrence 83-036/01T. This report is required within  
fourteen (14) days of the occurrence.

Sincerely yours,



J. M. Zupko, Jr.  
General Manager -  
Salem Operations

RF:k11 *gaj*

CC: Distribution