

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: | | | | | | | |

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

1 0 1 1 A R I A N O 1 0 1 2 1 0 1 0 1 0 1 0 1 0 1 0 1 3 1 4 1 1 1 1 1 1 4 1 1 1 5  
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

1 0 1 1 REPORT SOURCE 1 L 16 1 0 1 5 1 0 1 0 1 0 1 3 1 3 17 1 0 1 7 1 0 1 7 1 8 1 3 18 1 0 1 8 1 0 1 6 1 8 1 3 19  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

1. The Unit 1 once through steam generator (OTSG) had exhibited a small (approx. .007 gpm) primary to secondary leak since plant startup from a recent refueling outage. On 7/7/83, while the unit was at 100% full power, the leak slowly increased to approximately 0.1 gpm. The unit was shutdown for repair. During cooldown the leak rate increased to approximately 0.2 gpm as estimated by volumetric analysis. A total of 11.87 curies of activity was released; of this total, 11.85 curies were gaseous, predominately Xenon-133. No technical specification (T.S.) limits on releases or leak rates were exceeded. This occurrence is reportable per T.S. 16.12.3.2.d. Similar occurrences were reported in LER's (50-313) 78-005, 80-026, 80-034, and 82-012.

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE		COMP SUBCODE		VALVE SUBCODE		REVISION NO	
0   9		C   C   11		E   12		H   T   E   X   C   H   14		F   15		Z   16		0	
7 8		9 10		11		12 13 18		19		20		21	
17	LER/RO REPORT NUMBER	EVENT YEAR   1   8   3		---		SEQUENTIAL REPORT NO.   0   1   7		OCCURRENCE CODE   /		REPORT TYPE   L		REVISION NO   0	
		21   22		23		24 26 27		28 29		30 31		32	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		ATTACHMENT SUBMITTED		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
X   18		Z   19		A   20		A   21		Y   23		N   25		B   0   1   5   26	
33		34		35		36		37 40 41		42		43 44 47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS		27
1	1	0
1	1	1
1	1	2
1	1	3
1	1	4
7	8	9
FACILITY		METHOD

A single tube (77-18) was identified as the leaker. Eddy Current Testing confirmed the leak location to be at the interface of the tube and the upper OTSG tube sheet. The cause is believed to be intergranular stress corrosion cracking. The slight increase in leak rate during plant cooldown is consistent with increased OTSG tube tension and has been observed previously. An additional 533 steam generator tubes were inspected using eddy current methods. A total of 43 tubes were removed from service with plugs and upper end tube stabilizers.

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1   5   8 7   9   10	E   128   10   0   0   129   12	NA   13	A   131   44   45	Operator Observation   132   80
ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE	
1   6   8 7   9   10	M   133   M   134   11	11.87 curies, pred. X3-133   135	RCS to secondary to condenser to atmosphere   136	

PERSONNEL EXPOSURES				44	45	80
NUMBER		TYPE		DESCRIPTION		
1	7	12	0	8	37	E
138	Inspection/repairs 51.1 Man-Rem					
9	11	12	13	139		

PERSONNEL INJURIES										80	
NUMBER					DESCRIPTION						
1	8	0	0	0	40	NA					
8	9	11	12								14

LOSS OF OR DAMAGE TO FACILITY		TYPE DESCRIPTION		80	
1	9	1	2	42	NA

PUBLICITY		ISSUED		DESCRIPTION		NRC USE ONLY	
2	0	Y	44	News Release	145	1	1
8	9	10				68	69

NAME OF PREPARER Patrick C. Rogers

PHONE: (501) 964-3100

8308150072 830806  
PDR ADOCK 05000313  
S PDR

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LER No. 50-313/83-017/03L-0  
Occurrence Date: 07/07/83

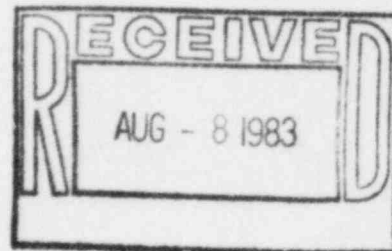
Cause Description and Corrective Actions (Continued)

Preliminary inspection results were discussed with the ANO-1 NRR Project Manager on July 15, 1983. Subsequently, more detailed inspection results were documented by letter 1CAN078312 on July 29, 1983. These inspection results and the results from the lab analysis of a previously pulled tube will be discussed in a future AP&L/NRC meeting.



ARKANSAS POWER & LIGHT COMPANY  
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

August 6, 1983



1CAN088303

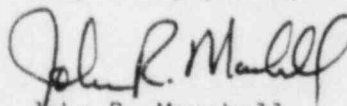
Mr. W. C. Seidle, Chief  
Reactor Project Branch #2  
U. S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, TX 76011

SUBJECT: Arkansas Nuclear One - Unit 1  
Docket No. 50-313  
License No. DPR-51  
Licensee Event Report  
No. 83-017/03L-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.d, attached is the subject report concerning a leak in the once through steam generator.

Very truly yours,

  
John R. Marshall  
Manager, Licensing

JRM:RJS:rd

Attachment

cc: Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Norman M. Haller, Director  
Office of Management & Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

IE-22