

NRC FORM 358
(7-77)

USNRC REGION II
ATLANTA, GEORGIA

U. S. NUCLEAR REGULATORY COMMISSION

CONTROL BLOCK: [] [] [] [] [] [] [] [] [] []

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | F I L M | P S 3 | 2 | 0 0 - 0 0 0 0 0 0 - 0 0 3 | 4 1 1 1 1 | 4 | | 5

7 3 9 14 15 25 26 30 37 38

LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO CAT

CCN'T

REPORT SOURCE: 0 1

60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

SOCKET NUMBER: L 5 0 1 0 2 5 0 7 0 9 1 7 8 1 2 8 0 7 2 9 8 3 9

EVENT DATE: 0 9 1 7 8 1 2 8 0 7 2 9 8 3 9

REPORT DATE: 0 7 2 9 8 3 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

012 In accordance with the Inservice Inspection and Testing Programs, a test was
013 being performed on auxiliary feedwater pumps A & C. C auxiliary feedwater
014 pump failed to run for the required 15 minutes. A and B pumps were in
015 satisfactory working order. This is reportable in accordance with
016 T.S.6.9.2.b.2. The health and safety of the public was not affected.
017 Similar events were reported as LERs 250-81-4, 250-79-36, 250-79-28, 250-79-34
018 250-79-17 and 250-74-7.

SYSTEM CODE H H		CAUSE CODE E		CAUSE SUBCODE B		COMPONENT CODE V A L V E X				COMP. SUBCODE F		VALVE SUBCODE H	
7	8	9	10	11	12	13	14	15	16	17	18	19	20
LER/RO REPORT NUMBER 8 2		EVENT YEAR 8 2		SEQUENTIAL REPORT NO. 0 1 3		OCCURRENCE CODE 0 3		REPORT TYPE X		REVISION NO. 1			
21	22	23	24	25	26	27	28	29	30	31	32		
ACTION TAKEN B		FUTURE ACTION F		EFFECT ON PLANT Z		SHUTDOWN METHOD Z		HOURS 0 0 0 0		ATTACHMENT SUBMITTED Y		APPROX. FORM SUB. Y	
33	34	35	36	37	38	39	40	41	42	43	44	45	46
PRIME COMP. SUPPLIER L		COMPONENT MANUFACTURER F 1 3 5											

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The steam turbine pressure control valve CV-3707 did not close completely.

1 1 This resulted in poor control of the steam pressure, causing the safety relief

1 2 valve to lift and the trip valve to close. Disassembly of CV-3707 revealed

1 3 two pieces of foreign metal under the valve seat. The valve was repaired

1 4 and its controls adjusted. The pump was returned to service in 44 hours.

FACILITY STATUS (1) 5 (E) (23) % POWER (1) 0 0 (29) OTHER STATUS (30) NA
 METHOD OF DISCOVERY (1) C (31) Inservice inspection DISCOVERY DESCRIPTION (32)
 ACTIVITY CONTENT RELEASED OF RELEASE (1) 5 (Z) (33) AMOUNT OF ACTIVITY (35) NA
 LOCATION OF RELEASE (36) NA

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION
1	7	0	0	NA

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	3	0	0
0	0	0	0
40	NA		

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1		2		3		4		5		6		7		8		9		10		11		12	
LOSS OF OR DAMAGE TO FACILITY (42)																							
TYPE		DESCRIPTION																					
1	9	Z	(42)	NA																			

3 2 10
PUBLICITY
ISSUED DESCRIPTION (45)
[2][0] [N] (41) NA _____ NAC USE ONLY

NRC USE ONLY

NAME OF PREPARER Z. E. Berry

PHONE: (305) 245-2910 ext 353

83 AUG 2 AIO: 27
Event Description and Probable Consequences

In accordance with the ASME Inservice Inspection and Testing Program, a test was being performed on the A and C auxiliary feedwater pumps. All three auxiliary feedwater pumps had successfully passed the monthly periodic test as required by T.S.4.10.1; however, in earlier tests, the developed pump head of pumps A and C had fallen in the alert range, requiring that the frequency of the test be doubled for those two pumps. On 9/17/82, the test was repeated for A and C pumps using OP 0209.3 (Inservice Pump Testing Program Implementation Procedure for Auxiliary Feedwater Pumps) and 7304.1 (Auxiliary Feedwater System - Periodic Test). C pump failed to run for the full 15 minutes. A and B pumps were in satisfactory working order. This is reportable in accordance with T.S.6.9.2.b.2. The health and safety of the public were not affected. Similar events were reported as LERs 251-81-4, 250-79-36, 250-79-34, 250-79-28, 250-79-17, and 250-74-7.

Additional Cause Description and Corrective Actions

The auxiliary steam turbine pressure control valve CV-3707 did not close completely. This resulted in poor control of the steam pressure, causing the 400 psi safety relief valve to lift and the trip valve to close. Disassembly of CV-3707 revealed two pieces of foreign metal under the valve seat, which were obstructing proper closure of the valve. Mechanical Maintenance replaced the gasket, teflon packing sets, valve plug, and valve stem. At this time, several minor repairs and adjustments were determined to be necessary and were made by Instrumentation and Control. The booster relay of differential pressure transmitter 2403 was replaced and the transmitter was calibrated. The AD relay of differential pressure controller 2403 was nulled and calibrated. The booster relay and proportional controller for pressure transmitter 3707 were replaced and the unit was calibrated. The integral unit for pressure comparator 3707 was replaced and calibrated. The valve positioner was calibrated. Adjustments were also made to the trip valve linkage. The pump was successfully retested and returned to service at 7:15 a.m. on 9/19/82. The pump had been considered inoperable for a total of 44 hours.

During the Unit 4 steam generator repair outage, new high pressure auxiliary feedwater pump turbines were installed. The steam pressure control valves are no longer required and have been removed from the system.

The source of the two pieces of foreign metal has not yet been determined. It is suspected that the source might be a piece of cage located inside the body of any one of twelve (six per unit) auxiliary feedwater pump steam-supply stop-check-valves upstream of CV-3707. These valves are normally locked in the open position and are used to isolate the steam-stop motor-operated-valves for maintenance. The present valves are manufactured by Walworth Company and replacement parts are no longer available. PC/Ms 82-311 and 82-312 are being written to evaluate alternatives and approve a new brand of replacement valve. Plant management has decided that disassembly and inspection of the present valve internals should be done as soon as the new valves are available and unit conditions permit. The six valves on each unit will be replaced during appropriate outages following approval and receipt of new valves.

83 AUG 2 AID: 27

Component Data

Control valve 3707 is a 4-inch globe valve manufactured by Fisher-Governor. The model number is 657 HS. Pressure comparator 3707 is an AD model, manufactured by Bailey. Pressure transmitter 3707 is a bourdon type with model number 4160, manufactured by Fisher Controls. Differential pressure controller 2403 is a Bailey model AD 52002. Differential pressure transmitter 2403 is a bellows indicating type with model number 225, manufactured by Barton.

USNRC REGION II
ATLANTA, GEORGIA

03 AUG 2 AIO: 01



July 29, 1983
PNS-LI-83-522-1

Mr. James P. O'Reilly
Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

REPORTABLE OCCURRENCE 250-82-13

TURKEY POINT UNIT 3

DATE OF OCCURRENCE: SEPTEMBER 17, 1982

TECHNICAL SPECIFICATION 6.9.2b.2

"C" AUXILIARY FEEDWATER PUMP

UPDATE REPORT NUMBER 1

The attached Licensee Event Report is being submitted to update our initial report dated October 18, 1982.

Very truly yours,

A handwritten signature in dark ink, appearing to read "J. W. Williams, Jr.", is written over a horizontal line.

J. W. Williams, Jr.
Vice President
Nuclear Energy

JWW/PLP/js

Attachment

cc: Director, Office of Inspection and Enforcement (40)
Harold F. Reis, Esquire
File 933.1 TP

OFFICIAL COPY

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