

NRC FORM 356
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

⑦ ⑧ ⑨ 0 1 F L C R P 3 ⑫ 2 0 0 - 0 0 0 0 0 - 0 0 ⑮ 3 4 1 1 1 1 ⑳ 4 ㉑ 5

LICENSE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

⑦ ⑧ ⑨ 0 1 L ⑫ 6 0 5 0 - 0 3 0 2 ⑮ 7 0 1 2 8 8 2 ⑲ 8 0 7 2 6 8 3 ㉓ 9

REPORT SOURCE 60 61 DOCKET NUMBER 65 66 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

⑦ ⑧ ⑨ 0 2 At 1430, during normal operation, RCS leakage calculations showed unidenti-

⑦ ⑧ ⑨ 0 3 fied leakage at greater than 1.0 gpm. The plant was manually shut down to

⑦ ⑧ ⑨ 0 4 Mode 3. At 1830 on January 29, 1982, while performing a visual inspection

⑦ ⑧ ⑨ 0 5 to identify leakage source, an RCS pressure boundary leak was discovered in

⑦ ⑧ ⑨ 0 6 the valve body of MUV-43. These events were contrary to T.S. 3.4.6.2. The

⑦ ⑧ ⑨ 0 7 plant was cooled down to Mode 5. This is the first occurrence concerning

⑦ ⑧ ⑨ 0 8 MUV-43 and the second report under this Specification.

⑦ ⑧ ⑨ 0 9 S F ⑪ B ⑫ A ⑬ V A L V E X ⑮ C ⑯ A ⑰

SYSTEM CODE 9 10 CAUSE CODE 11 CAUSE SUBCODE 12 COMPONENT CODE 13 COMP. SUBCODE 14 VALVE SUBCODE 15

⑦ ⑧ ⑨ 1 7 8 2 ㉑ 0 0 3 ㉒ 0 1 ㉓ T ㉔ 1 ㉕

LER/RO REPORT NUMBER 21 22 EVENT YEAR 23 24 SEQUENTIAL REPORT NO. 25 26 OCCURRENCE CODE 27 28 REPORT TYPE 29 30 REVISION NO. 31 32

⑦ ⑧ ⑨ 1 5 E ㉖ 0 8 4 ㉗ N/A ㉘ A ㉙ Operator Observation ㉚

FACILITY STATUS 28 29 % POWER 30 31 OTHER STATUS 32 33 METHOD OF DISCOVERY 34 35 DISCOVERY DESCRIPTION 36 37

⑦ ⑧ ⑨ 1 6 Z ㉛ Z ㉜ N/A ㉝ N/A ㉞

ACTIVITY CONTENT 33 34 RELEASED OF RELEASE 35 36 AMOUNT OF ACTIVITY 37 38 LOCATION OF RELEASE 39 40

⑦ ⑧ ⑨ 1 7 0 0 0 ㉟ Z ㊱ N/A ㊲

PERSONNEL EXPOSURES NUMBER 37 38 TYPE 39 40 DESCRIPTION 41 42

⑦ ⑧ ⑨ 1 8 0 0 0 ㊲ N/A ㊳

PERSONNEL INJURIES NUMBER 40 41 TYPE 42 43 DESCRIPTION 44 45

⑦ ⑧ ⑨ 1 9 Z ㊴ N/A ㊵

LOSS OF OR DAMAGE TO FACILITY TYPE 42 43 DESCRIPTION 44 45

⑦ ⑧ ⑨ 2 0 Y ㊶ Press release made denoting plant shutdown. ㊷

ISSUED DESCRIPTION 44 45

⑦ ⑧ ⑨ 2 0 Y ㊶ Press release made denoting plant shutdown. ㊷

PUBLICITY 45 46 NRC USE ONLY 47 48

NAME OF PREPARER Charles G. Brown PHONE: (904) 795-6486

SUPPLEMENTARY INFORMATION

REPORT NO: 50-302/82-003/01T-1
FACILITY: Crystal River Unit #3
REPORT DATE: July 26, 1983
OCCURRENCE DATE: January 29, 1982

IDENTIFICATION OF OCCURRENCE:

A Reactor Coolant pressure boundary leak was discovered in MUV-43, creating an event contrary to Technical Specification 3.4.6.2.

CONDITIONS PRIOR TO OCCURRENCE:

MODE 1 (84% FULL POWER).

DESCRIPTION OF OCCURRENCE:

While performing a visual inspection of MUV-43 for a possible bonnet leak, a circumferential crack was discovered in the valve body, adjacent to the safe end weld on MUV-43, with some through-wall failure. MUV-43 was removed for failure analysis.

DESIGNATION OF APPARENT CAUSE:

The cause of this event is attributed to thermally induced cyclic fatigue.

ANALYSIS OF OCCURRENCE:

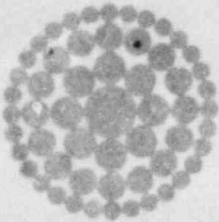
There was no effect upon the health or safety of the general public.

CORRECTIVE ACTION:

MUV-43 and related piping was replaced. Related areas of all four (4) injection lines were inspected to assess the scope of the problem. A detailed report describing Florida Power Corporation's investigations and repairs was submitted to Mr. J. P. O'Reilly on April 30, 1982.

FAILURE DATA:

This is the first occurrence concerning MUV-43, and this is the second event reported under this Specification.



USNRC REG
ATLANTA, GE

83 JUL 29 A 9

**Florida
Power**
CORPORATION

July 26, 1983
3F-0783-21

Mr. James P. O'Reilly
Regional Administrator, Region II
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, GA 30303

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Licensee Event Report No. 82-003

Dear Mr. O'Reilly:

Enclosed is Licensee Event Report No. 82-003 and the attached supplementary information sheet, which are submitted in accordance with Technical Specification 6.9.1.8.i. This report supplies supplementary information to our initial report dated February 12, 1982.

Should there be any questions, please contact this office.

Sincerely,

G. R. Westafer
Manager
Nuclear Licensing and Fuel Management

AEF:mm

Enclosure

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

OFFICIAL COPY

IE 22