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ELECTRIC ENGINEERING
DEPARTMENT

July 1, 1983

Director of Nuclear Reactor Regulation
Attention: Mr. R. A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Calvert Cliffs Nuclear Power Plant
Units Nos. 1 & 2; Dockets Nos. 50-317 and 50-318
Main Steam Line Break Inside Containment

Gentlemen:

On June 23, 1983 you requested a statistical evaluation of the actual material strengths for containment tendons. You stated that this information was needed in order to complete your evaluation of issues related to I&E Bulletin 80-04.

In response to your request, please be informed that in a post-tensioned containment design, the critically-stressed components during containment over-pressurization would be the reinforcing steel (rebar). Significant containment loads will not be transferred to the tendons until after the reinforcing steel approached or began to exceed its yield strength.

Since it has been demonstrated that the yield strength of reinforcing steel will not be exceeded for a main steam line break an evaluation of actual tendon material strengths is not necessary. Such an evaluation would only be pertinent in an assessment of ultimate containment capacity.

If you should have any questions, please do not hesitate to contact us.

Very truly yours,

Bruce S. Montgomery

BSM/vf

cc: J. A. Biddison, Jr., Esq.
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Mr. D. H. Jaffe, NRC
Mr. R. E. Architzel, NRC

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