

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)0 1 P A S E S 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 9 14 15 25 26 30 37 38 39

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 8 7 7 0 5 3 0 8 3 8 0 6 2 9 8 3 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During testing, the Reactor Core Isolation Cooling (RCIC) pump discharge valve
0 3 would not open. The Limiting Condition for Operation (LCO) 3.7.3 was declared
0 4 and investigation of the problem initiated. The Technical Specification Action
0 5 Statement was followed and there were no consequential effects to public health
0 6 and safety.

0 7

0 8

0 9 C E 11 E 12 A 13 C K T B R K 14 A 15 Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20

17 LER/RO REPORT NUMBER 8 3 21 22
18 ACTION TAKEN B 18 19 FUTURE ACTION X 19 20 EFFECT ON PLANT Z 20 21 SHUTDOWN METHOD Z 21 22 HOURS 0 0 0 0 22 23 ATTACHMENT SUBMITTED N 23 24 NPRD-4 FORM SUB. N 24 25 PRIME COMP. SUPPLIER N 25 26 COMPONENT MANUFACTURER G 0 8 0 26
33 34 35 36 37 38 39 40 41 42 43 44 45

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Maintenance personnel performed a series of electrical checks and determined a gap
1 1 existed between the breaker male and female stab clips. The female clips were re-
1 2 worked to allow proper contact. Testable safety related loads with similar elect-
1 3 rical devices were verified operable and further investigation is being done to
1 4 determine appropriate corrective action to prevent recurrence.

1 5 B 28 0 9 0 29 NA 30 B 31 Operator Observation 32
7 8 9 10 11 12 13 14 15 16 17 18 19 201 6 Z 33 Z 34 NA 35 NA 36
7 8 9 10 11 12 13 14 15 16 17 18 19 201 7 0 0 0 37 Z 38 NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 201 8 0 0 0 40 NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 201 9 Z 42 NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 202 0 N 44
7 8 9 10 11 12 13 14 15 16 17 18 19 20

NAME OF PREPARER A.P. Piemontese

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June 29, 1983

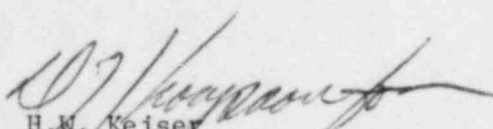
Mr. J.M. Allan
Acting Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 83-088/03L-0
ER 100450 FILE 841-23
PLA-1736

Docket No. 50-387
License No. NPF-14

Dear Mr. Allan:

Attached please find a copy of Licensee Event Report No. 83-088/03L-0. This event was determined to be reportable per Technical Specification 6.9.1.9.b, in that the Reactor Core Isolation Cooling (RCIC) pump discharge valve failed to open during testing. Technical Specification action statements were complied with and the valve has subsequently been returned to operable status.


H.W. Keiser
Superintendent of Plant-Susquehanna

APP/pjg

cc: G.G. Rhoads
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