

3150-0011

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

PHONE (205) 729-0893

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LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83030 Technical Specification Involved 3.7.D.1

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 07/07/83

Event Narrative:

Unit 1 was shut down for a refueling outage; unit 3 was operating at 99-percent power. These units were unaffected by this event. While unit 2 was operating normally at 93-percent power, a differential pressure check was made during operation of the drywell differential air compressor and FCV-64-34, torus exhaust valve bypass to standby gas treatment system, failed to meet its' five-second maximum operating time (Technical Specification 3.7.D.2 and Table 3.7.A.) FCV-64-31, 139, and 141 were closed to permit continued reactor power operation. In an accident situation, FCV-64-34 would have isolated. An urgent maintenance request was written to investigate and repair. FCV-64-34 was found to be operable; however, the limit switches which give control room indication were out of adjustment. The limit switches were adjusted to indicate proper valve position and satisfactorily tested. There was no effect on public health or safety. Redundant valves FCV-64-139, 64-140, and 64-31 were available and operable. This is considered a random event and no recurrence control is required.

* Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

REGION II
TENNESSEE VALLEY AUTHORITY

ATLANTA, GEORGIA
CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

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July 6, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260/83030

The enclosed report provides details concerning a torus exhaust valve
bypass to standby gas treatment which failed to operate within required
time limits. This report is submitted in accordance with Browns Ferry
unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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