NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

Update Report:
Previous report date: 5-25-83

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 — LICENSEE CODE: N C B E P 1 (2) — LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) — LICENSE TYPE: 4 1 1 1 1 (4) — CAT (5)

CON'T 0 1 — REPORT SOURCE: L (6) — DOCKET NUMBER: 0 5 0 - 0 3 2 5 (7) — EVENT DATE: 0 4 2 5 8 3 (8) — REPORT DATE: 0 6 3 0 8 3 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During unit refueling operations, performance of the SLCS relief valve operability
0 3 test, PT-06.2.1, revealed SLCS relief valves 1-C41-F029A and F029B lifted at 1321
0 4 psig and 1592 psig, respectively. On April 27, 1983, performance of the PT on Unit
0 5 No. 2 revealed SLCS relief valve 2-C41-F029A lifted at 1403 psig while F029B lifted at
0 6 1323. The required lifting setpoint of SLCS relief valves is 1400 ± 50 psig. Neither
0 7 of these events affected the health and safety of the public.
0 8 Technical Specifications 3.1.5, 6.9.1.9b

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	R B (11)	D (12)	Z (13)	V A L V E X (14)	X (15)	B (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 1 9	/	0 3	L	—	1

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	N (25)	L 2 6 5 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The out-of-tolerance relief valve liftings are attributed to adjustment of the
1 1 valves' relief setpoints using the SLCS pumps discharge pressure as a test medium
1 2 and audible report of valve opening or line flow to verify the setpoint. In accor-
1 3 dance with a March 9, 1983, revision of the periodic test, the valves on both units
1 4 were adjusted and satisfactorily tested. No further action regarding these events
is planned.

1 5 — FACILITY STATUS: H (28) — % POWER: 0 0 0 (29) — OTHER STATUS (30): NA — METHOD OF DISCOVERY: (31) — DISCOVERY DESCRIPTION (32): Periodic Text

1 6 — ACTIVITY RELEASED: Z (33) — CONTENT OF RELEASE: Z (34) — AMOUNT OF ACTIVITY (35): NA — LOCATION OF RELEASE (36): NA

1 7 — PERSONNEL EXPOSURES — NUMBER: 0 0 0 (37) — TYPE: Z (38) — DESCRIPTION (39): NA

1 8 — PERSONNEL INJURIES — NUMBER: 0 0 0 (40) — DESCRIPTION (41): NA

1 9 — LOSS OF OR DAMAGE TO FACILITY (43) — TYPE: Z (42) — DESCRIPTION: NA

2 0 — PUBLICITY — ISSUED: N (44) — DESCRIPTION (45): A — NRC USE ONLY

NAME OF PREPARER: M. J. Pastva, Jr. PHONE: 457-9521 (919)

GPO 917-926

LER ATTACHMENT - RO #1-83-19

Facility: Unit No. 1 Event Date: April 25, 1983

The Unit No. 1 performance of SLCS relief valve operability test, PT-06.2.1, revealed relief valves 1-C41-F029A and F029B lifted at 1321 psig and 1592 psig, respectively. On April 27, 1983, performance of the periodic test (PT) on Unit No. 2 revealed SLCS relief valve F029B lifted at 1323 psig. Technical specifications require the lifting setpoints of SLCS relief valves to be 1400 ± 50 psig.

The out-of-tolerance relief valve setpoints are attributed to adjustment of the valves during the performance of the PT prior to March 9, 1983. At that time, in accordance with the PT, the SLCS pumps' discharge pressure was used as the sole pressure medium and the audible report of valve opening and/or line flow was used as verification of the valve setpoint. On March 9, 1983, the PT had been revised to provide for adjustment of the SLCS relief valve setpoints using one of the two following methods:

1. Removal of the valves from the system for bench testing and adjustment as required.

2. Testing the valves as installed in the SLCS and performing any required adjustments using a hydraulic assist method to ensure an accurate test medium pressure.

It is felt performance of the revised PT will help in the prevention of future similar events; therefore, further action regarding this event is not planned.

CP&L

Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

June 30, 1983

FILE: B09-13510C
SERIAL: BSEP/83-2010

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street NW
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
SUPPLEMENT TO LICENSEE EVENT REPORT 1-83-19

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within thirty (30) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

J. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/shb/LETSB1

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk