

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/6/1/4/8/3/ (8) /0/6/2/2/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On June 14, 1983, with Unit 2 at 100% power, Fire Door S71-18 between the "2H" /
/0/3/ / Emergency Diesel Generator Room and the Turbine Building would not latch and /
/0/4/ / lock. Since a fire watch was immediately posted the public health and safety /
/0/5/ / were not affected. This event is contrary to T.S. 3.7.15 and reportable to T.S. /
/0/6/ / 6.9.1.9.b. Degradation of fire doors between the Emergency Diesel Room and the /
/0/7/ / Turbine Building is a recurring event due to the differential pressure during /
/0/8/ / diesel operation. /

SYSTEM CAUSE CAUSE COMP. VALVE
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /A/B/ (11) /E/ (12) /B/ (13) /X/X/X/X/X/X/ (14) /Z/ (15) /Z/ (16)
LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION
(17) REPORT NO. NO.
NUMBER /8/3/ /-/ /0/4/5/ / / /0/3/ /L/ /-/ /0/

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/E/ (18) /X/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /N/ (24) /A/ (25) /C/1/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Fire Door S71-18 would not latch because the latch was sticking. The latch was /
/1/1/ / adjusted, lubricated and the operability of the fire door verified. Design /
/1/2/ / modifications to the fire doors between the Emergency Diesel Room and the Turbine /
/1/3/ / Building are being pursued. /
/1/4/ / /

FACILITY METHOD OF
STATUS %POWER OTHER STATUS DISCOVERY DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28) /1/0/0/ (29) / NA / (30) /A/ (31) / Operator Observation /

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES
NUMBER DESCRIPTION (41)
/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
/1/9/ /Z/ (42) / NA /

PUBLICITY
ISSUED DESCRIPTION (45)
/2/0/ /N/ (44) / NA /

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

8307120254 830622
PDR ADDCK 05000359
S PDR

Vepco

USNRC REGION II
ATLANTA, GEORGIA

83 JUN 29 9:22
VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 23117

June 22, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-087
NO/CLF: 11
Docket No. 50-339
License No. NPF-7

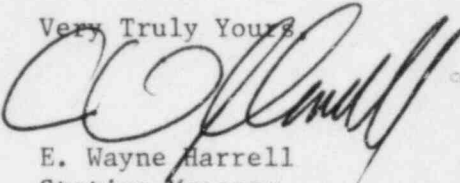
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.	Applicable Technical Specifications
LER 83-045/03L-0	T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours


E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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Washington, D. C. 20555

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