

3150-0011

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTEVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

REVIS
NO
- | 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 4 will be issued by November 1, 1983.

PHONE (205) 729-0621

8307120091 830701
PDR ADCK 05000260
S PDR

NRC USE ONLY

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83031 Technical Specification Involved 3.6.C.2

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 7/05/83

Event Narrative:

Unit 1 was in a refueling outage; unit 3 was operating at 69-percent power and both of these units were unaffected by this event. During startup on unit 2 with the unit operating at 87-percent power, annunciator RA-90-256A (drywell leak detection radiation HI) and RA-90-256E (drywell leak detection rad downscale) were received on panel 9-3. The sample pump for drywell radiation monitor RM-90-256 was found to be tripped due to a blown 15 amp supply fuse. When the Bussman Part No. MIC-15 fuse was replaced, the iodine channel of the CAM was found to be inoperable. The high voltage of the iodine channel was adjusted, a functional SI was performed on the CAM, and the drywell air sampling system was returned to service. The drywell air sampling system was inoperable for approximately 16 hours. The sump sampling system was operable. Technical Specification 3.6.C.2 permits operation for seven days with the air sampling system inoperable. No grab samples were taken as the monitor was returned to service within 24-hours (TS 3.6.C.2.) There was no effect on public health or safety. Investigation did not reveal an apparent cause for drift of the high voltage adjustment. Maintenance Request A-062737 has been issued to investigate this case to determine if the fuse failure was a random occurrence or if other problems exist. Any problems found will be corrected, and a followup report will be issued by November 1, 1983.

* Previous Similar Events:

BFRO-50-260/83029
296/82063

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

USNRC REGION 1
ATLANTA, GEORGIA
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

83 JUL 7 09:33

July 1, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260/83031

The enclosed report provides details concerning a drywell leak detection
radiation monitor that became inoperable. This report is submitted in
accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. J. Green

H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

OFFICIAL COPY

IE 22
1/1